

May 28, 2010

LICENSEE: PSEG Nuclear, LLC

FACILITY: Hope Creek Generating Station, Units 1 and 2

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON  
MARCH 4, 2010, BETWEEN THE U.S. NUCLEAR REGULATORY  
COMMISSION AND PSEG NUCLEAR, LLC, CONCERNING DRAFT  
REQUESTS FOR ADDITIONAL INFORMATION PERTAINING TO THE  
HOPE CREEK GENERATING STATION, LICENSE RENEWAL APPLICATION

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of PSEG Nuclear, LLC (the applicant), and Exelon held a telephone conference call on March 4, 2010, to discuss and clarify the staff's draft requests for additional information (D-RAs) concerning the Hope Creek Generating Station, license renewal application. The telephone conference call was useful in clarifying the intent of the staff's D-RAs.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a listing of the D-RAs discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary.

/RA/

Bennett M. Brady, Project Manager  
Projects Branch 1  
Division of License Renewal  
Office of Nuclear Reactor Regulation

Docket Nos. 50-354

Enclosures:

1. List of Participants
2. List of Draft Requests for  
Additional Information

cc w/encls: See next page

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TELEPHONE CONFERENCE CALL  
HOPE CREEK GENERATING STATION  
LICENSE RENEWAL APPLICATION

LIST OF PARTICIPANTS  
MARCH 4, 2010

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Raman Pichumani  
Juan Uribe  
George Seibold  
Jim Stavely  
Don Warfel  
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PSEG Nuclear, LLC (PSEG)  
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Exelon  
Exelon

DRAFT REQUESTS FOR ADDITIONAL INFORMATION  
HOPE CREEK GENERATING STATION  
LICENSE RENEWAL APPLICATION

MARCH 4, 2010

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of PSEG Nuclear, LLC held a telephone conference call on March 4, 2010, to discuss and clarify the following draft requests for additional information (D-RAIs) concerning the Hope Creek Generating Station (HCGS), license renewal application (LRA).

**D-RAI 2.4-1** Which components, if any, in the HCGS LRA Table 2.4.3 (Auxiliary Building Service / Radwaste Area) include the main steam tunnel structural elements? If these are omitted, please justify their omission.

**Discussion:** The applicant understands the question and stated that the elements have not been omitted and will provide the response.

**D-RAI 2.4-2** HCGS LRA Section 2.4.4 (Component Supports Commodity Group) states that snubbers are also included in the boundary of this commodity group; however they are considered active components and are not subject to an aging management review (AMR), except for the end connections, which perform a passive function for structural support. This statement implies that the end connections are subject to an AMR. Identify in which category are these end connections included in the HCGS LRA Table 2.4.4.

**Discussion:** The applicant stated that the snubbers are included in the bolted connections section. The reviewer understands the response and agrees to withdraw the question and renumber the remaining questions.

**D-RAI 2.4-3** Reinforced concrete isolation walls are included in the LRA Table 2.4.2 (Auxiliary Building Control/Diesel Generator Area); however, the isolation function is not listed as an intended function in any concrete component listed in the aforementioned table. Please clarify if this intended function is listed as a different function (shielding, structural support, etc) or justify its exclusion.

**Discussion:** The applicant understands the question. The question will be renumbered RAI 2.4-2.

**D-RAI 2.4-4** Horizontal seismic restraints are shown in Figures 3.8.1 and 3.8.13 of the HCGS UFSAR for the Torus of the Primary Containment. These are not included in Table 2.4.7 of the HCGS LRA Section 2.4.7 (Primary Containment). Justify their exclusion.

**Discussion:** The applicant understands the question. The question will be renumbered RAI 2.4-3.

**D-RAI 2.4-5** Please indicate where the structural elements that transfer the reactor pressure vessel (RPV) loads to the RPV ring girder and subsequently to the RPV pedestal (shown in Figure 3.8.1 in HCGS UFSAR) are evaluated in the HCGS LRA. If they have not been evaluated, please justify their exclusion.

**Discussion:** The applicant understands the question. The question will be renumbered RAI 2.4-4.

**D-RAI 2.4-6** HCGS LRA Table 2.4.7 (Primary Containment) lists the torus ring girder as one of the components subject an AMR. But this table does not include the RPV ring girder, as shown in UFSAR Figure. 3.8.1. Please justify the omission of the latter.

**Discussion:** The applicant understands the question. The question will be renumbered RAI 2.4-5.

**D-RAI 2.4-7** Based on a review of the HCGS LRA Sections 2.4.7 (Primary Containment) and 2.4.8 (Reactor Building) and of Tables 2.4-7 and 2.4.8, it is not clear if the following components have been included in the scope of license renewal and subject to an AMR:

- a) Refueling seal assembly
- b) Weld pads on the drywell shell for attachment of pipe supports
- c) Water seal plates at the base of the drywell head, as shown in UFSAR Figure 3.8.1
- d) Spent fuel pool liner plate leak chase system

If these components are covered somewhere else in the LRA, please indicate the location. If they are excluded from the scope of license renewal, please provide the basis for their exclusion.

**Discussion:** The applicant understands the question. The question will be renumbered RAI 2.4-6.

**D-RAI 2.4-8** UFSAR Figure 3.8-29 (Biological Shield Plan and Elevation) shows details of shear ties. Please identify the component in LRA Table 2.4.7, which includes these structural elements If these components are covered somewhere else in the LRA, please indicate the location. If they are excluded from the scope of license renewal, please provide the basis for their exclusion.

**Discussion:** The applicant understands the question. The question will be renumbered RAI 2.4-7.

Memorandum to PSEG Nuclear, LLC from B. Brady, dated May 28, 2010

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