



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

May 25, 2010

Vice President, Operations  
Arkansas Nuclear One  
Entergy Operations, Inc.  
1448 S.R. 333  
Russellville, AR 72802

SUBJECT: ARKANSAS NUCLEAR ONE, UNIT NO. 2 - ISSUANCE OF AMENDMENT  
NO. 290 RE: LICENSE AMENDMENT REQUEST TO REVISE TECHNICAL  
SPECIFICATION 6.6.5, "CORE OPERATING LIMITS REPORT (COLR)" (TAC  
NO. ME1329)

Dear Sir or Madam:

The U.S. Nuclear Regulatory Commission (NRC) has issued the enclosed Amendment No. 290 to Renewed Facility Operating License No. NPF-6 for Arkansas Nuclear One, Unit No. 2 (ANO-2). The amendment consists of changes to the Technical Specifications (TSs) in response to your application dated May 15, 2009, as supplemented by letters dated November 10, 2009, and May 17, 2010.

The amendment modifies TS 6.6.5, "Core Operating Limits Report (COLR)," to minimize the number of NRC-approved references consistent with the guidance provided in NRC Generic Letter 88-16, "Removal of Cycle-Specific Parameter Limits from Technical Specifications," dated October 3, 1988. This also fulfills the commitment made in the licensee's letter to the NRC dated March 11, 2008, "Response to Request for Additional Information, License Amendment Request to Revise Technical Specification 6.6.5, Core Operating Limits Report."

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "N. Kaly Kalyanam", is positioned below the word "Sincerely,".

N. Kaly Kalyanam, Project Manager  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-368

Enclosures:

1. Amendment No. 290 to NPF-6
2. Safety Evaluation

cc w/encs: Distribution via Listserv



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

ENTERGY OPERATIONS, INC.

DOCKET NO. 50-368

ARKANSAS NUCLEAR ONE, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 290  
Renewed License No. NPF-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Entergy Operations, Inc. (the licensee), dated May 15, 2009, as supplemented by letters dated November 10, 2009, and May 17, 2010, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 1

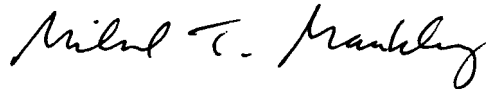
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-6 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 290, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications

3. The license amendment is effective as of its date of issuance and shall be implemented within 90 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael T. Markley, Chief  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Renewed Facility  
Operating License No. NPF-6  
Technical Specifications

Date of Issuance: May 25, 2010

ATTACHMENT TO LICENSE AMENDMENT NO. 290

RENEWED FACILITY OPERATING LICENSE NO. NPF-6

DOCKET NO. 50-368

Replace the following pages of the Renewed Facility Operating License No. NPF-6 and Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Operating License

REMOVE

-3-

INSERT

-3-

Technical Specifications

REMOVE

6-20

6-21

INSERT

6-20

6-21

- (4) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (5) EOI, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (6) EOI, pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to conditions specified in the following Commission regulations in 10 CFR Chapter 1; Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

EOI is authorized to operate the facility at steady state reactor core power levels not in excess of 3026 megawatts thermal. Prior to attaining this power level EOI shall comply with the conditions in Paragraph 2.C.(3).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 290 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

Exemptive 2nd paragraph of 2.C.2 deleted per Amendment 20, 3/3/81.

(3) Additional Conditions

The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following issuance of the renewed license or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the renewed license supported by a favorable evaluation by the Commission.

2.C.(3)(a) Deleted per Amendment 24, 6/19/81.

## ADMINISTRATIVE CONTROLS

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### 6.6.5 CORE OPERATING LIMITS REPORT (COLR)

- a. Core operating limits shall be established prior to each reload cycle, or prior to any remaining part of a reload cycle, and shall be documented in the COLR for the following:
  - 3.1.1.1 Shutdown Margin –  $T_{avg} > 200^{\circ}\text{F}$
  - 3.1.1.2 Shutdown Margin –  $T_{avg} \leq 200^{\circ}\text{F}$
  - 3.1.1.4 Moderator Temperature Coefficient
  - 3.1.3.1 CEA Position
  - 3.1.3.6 Regulating and Group P CEA Insertion Limits
  - 3.2.1 Linear Heat Rate
  - 3.2.3 Azimuthal Power –  $T_q$
  - 3.2.4 DNBR Margin
  - 3.2.7 Axial Shape Index
- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:
  - 1) "Qualification of the PHOENIX-P/ANC Nuclear Design System for Pressurized Water Reactor Cores" (WCAP-11596-P-A), "ANC: A Westinghouse Advanced Nodal Computer Code" (WCAP-10965-P-A), and "ANC: A Westinghouse Advanced Nodal Computer Code: Enhancements to ANC Rod Power Recovery" (WCAP-10965-P-A Addendum 1) (Methodology for Specifications 3.1.1.1 and 3.1.1.2 for Shutdown Margins, 3.1.1.4 for MTC, 3.1.3.6 for Regulating and Group P CEA Insertion Limits, and 3.2.4.b for DNBR Margin).
  - 2) "CE Method for Control Element Assembly Ejection Analysis," CENPD-0190-A (Methodology for Specification 3.1.3.6 for Regulating and Group P CEA Insertion Limits and 3.2.3 for Azimuthal Power Tilt).
  - 3) "Modified Statistical Combination of Uncertainties, CEN-356(V)-P-A, Revision 01-P-A (Methodology for Specification 3.2.4.c and 3.2.4.d for DNBR Margin and 3.2.7 for ASI).
  - 4) "Calculative Methods for the CE Large Break LOCA Evaluation Model," CENPD-132-P (Methodology for Specification 3.1.1.4 for MTC, 3.2.1 for Linear Heat Rate, 3.2.3 for Azimuthal Power Tilt, and 3.2.7 for ASI).
  - 5) "Calculative Methods for the CE Small Break LOCA Evaluation Model," CENPD-137-P (Methodology for Specification 3.1.1.4 for MTC, 3.2.1 for Linear Heat Rate, 3.2.3 for Azimuthal Power Tilt, and 3.2.7 for ASI).

## ADMINISTRATIVE CONTROLS

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### 6.6.5 CORE OPERATING LIMITS REPORT (COLR) (Continued)

- 6) "Technical Manual for the CENTS Code," WCAP-15996-P-A, Rev. 1 (Methodology for Specifications 3.1.1.1 and 3.1.1.2 for Shutdown Margin, 3.1.1.4 for MTC, 3.1.3.1 for CEA Position, 3.1.3.6 for Regulating and Group P Insertion Limits, and 3.2.4.b for DNBR Margin).
  - 7) "Implementation of ZIRLO Material Cladding in CE Nuclear Power Fuel Assembly Designs," CENPD-404-P-A (modifies CENPD-132-P and CENPD-137-P as methodology for Specification 3.1.1.4 for MTC, 3.2.1 for Linear Heat Rate, 3.2.3 for Azimuthal Power Tilt, and 3.2.7 for ASI).
  - 8) "Qualification of the Two-Dimensional Transport Code PARAGON," WCAP-16045-P-A (may be used as a replacement for the PHOENIX-P lattice code as the methodology for Specifications 3.1.1.1 and 3.1.1.2 for Shutdown Margins, 3.1.1.4 for MTC, 3.1.3.6 for Regulating and Group P CEA Insertion Limits, and 3.2.4.b for DNBR Margin).
  - 9) "Implementation of Zirconium Diboride Burnable Absorber Coatings in CE Nuclear Power Fuel Assembly Designs," WCAP-16072-P-A (Methodology for Specification 3.1.1.4 for MTC, 3.2.1 for Linear Heat Rate, 3.2.3 for Azimuthal Tilt, and 3.2.7 for ASI).
  - 10) "CE 16 x 16 Next Generation Fuel Core Reference Report," WCAP-16500-P-A (Methodology for Specification 3.1.1.4 for MTC, 3.2.1 for Linear Heat Rate, 3.2.3 for Azimuthal Power Tilt, 3.2.4.b, 3.2.4.c and 3.2.4.d for DNBR Margin, and 3.2.7 for ASI).
  - 11) "Optimized ZIRLO™," WCAP-12610-P-A and CENPD-404-P-A Addendum 1-A (Methodology for Specification 3.1.1.4 for MTC, 3.2.1 for Linear Heat Rate, 3.2.3 for Azimuthal Power Tilt, and 3.2.7 for ASI).
  - 12) "Westinghouse Correlations WSSV and WSSV-T for Predicting Critical Heat Flux in Rod Bundles with Side-Supported Mixing Vanes," WCAP-16523-P-A (Methodology for Specification 3.2.4.b, 3.2.4.c and 3.2.4.d for DNBR Margin).
  - 13) "ABB Critical Heat Flux Correlations for PWR Fuel," CENPD-387-P-A (Methodology for Specification 3.2.4.b, 3.2.4.c and 3.2.4.d for DNBR Margin and 3.2.7 for ASI).
- c. The core operating limits shall be determined such that all applicable limits (e.g. fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling System (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
  - d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 290 TO

RENEWED FACILITY OPERATING LICENSE NO. NPF-6

ENTERGY OPERATIONS, INC.

ARKANSAS NUCLEAR ONE, UNIT NO. 2

DOCKET NO. 50-368

1.0 INTRODUCTION

By application dated May 15, 2009 (Reference 1), as supplemented by letters dated November 10, 2009, and May 17, 2010 (References 2 and 3, respectively), Entergy Operations, Inc. (the licensee), requested changes to the Technical Specifications (TSs) for Arkansas Nuclear One, Unit No. 2 (ANO-2). The supplemental letters dated November 10, 2009, and May 17, 2010, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on June 30, 2009 (74 FR 31321).

The proposed changes would revise TS 6.6.5, "Core Operating Limits Report (COLR)," to minimize the number of U.S. Nuclear Regulatory Commission (NRC)-approved references consistent with the guidance provided in NRC Generic Letter (GL) 88-16, "Removal of Cycle-Specific Parameter Limits from Technical Specifications," dated October 3, 1988. This would also fulfill the commitment made in the licensee's letter to the NRC dated March 11, 2008, "Response to Request for Additional Information License Amendment Request to Revise Technical Specification 6.6.5, Core Operating Limits Report" (Reference 4).

2.0 REGULATORY EVALUATION

In Section 50.36, "Technical specifications," of Title 10 of the *Code of Federal Regulations* (10 CFR 50.36), the Commission established its regulatory requirements related to the content of the TSs. Pursuant to 10 CFR 50.36, TSs are required to include items in the following five specific categories related to station operation: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCOs); (3) surveillance requirements; (4) design features; and (5) administrative controls. The rule does not specify the particular requirements to be included in a plant's TSs. As stated in 10 CFR 50.36(c)(2)(i), the "[l]imiting conditions for operation are the lowest functional capability or performance levels of



equipment required for safe operation of the facility. When a limiting condition for operation of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the technical specifications ...”

The regulations in 10 CFR 50.36(c)(2)(ii) state that a TS LCO of a nuclear reactor must be established for each item meeting one or more of the following criteria:

*Criterion 1.* Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.

*Criterion 2.* A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

*Criterion 3.* A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

*Criterion 4.* A structure, system, or component which operating experience or probabilistic risk assessment has shown to be significant to public health and safety.

License amendments are generally required for each fuel cycle to update the values of cycle-specific parameter limits in TSs. However, to eliminate the need for a license amendment to update the cycle-specific parameter limits for each fuel cycle while meeting 10 CFR 50.36(c)(2)(ii) requirements, the NRC staff has allowed licensees to use an alternative to incorporate the cycle-specific parameter limits in the COLR.

Guidance on the relocation of cycle-specific TS parameters to the COLR is provided to all power reactor licensees and applicants in NRC GL 88-16, “Removal of Cycle-Specific Parameter Limits from Technical Specifications,” dated October 3, 1988. In GL 88-16, the NRC staff stated that license amendments are generally required every refueling outage to update the cycle-specific parameter limits in the TSs; however, there are methodologies developed for the licensee to determine these cycle-specific parameters that have been reviewed and approved by the staff. As a consequence, the NRC staff review of proposed changes to the TSs to update these parameter limits is primarily limited to the confirmation that the updated limits were calculated by the approved methodology and consistent with the appropriate plant-specific safety analysis. The COLR was created to place the NRC-approved methodologies in the TSs and allow licensees to use later revisions of these methodologies to update the parameters without requiring a change to the TSs.

### 3.0 TECHNICAL EVALUATION

#### 3.1 Deletion of Parenthetical COLR Parameter-to-Methodology Cross-References

In its application dated May 15, 2009, the licensee proposed to delete the parenthetical COLR parameter-to-methodology cross-references. The NRC staff questioned the appropriateness, in part, because without the COLR parameter-to-methodology cross-references, the qualification for the approved methodology to be listed in TS 6.6.5.b would be in question. In its supplemental letter dated May 17, 2010, the licensee agreed to retain the parenthetical COLR parameter-to-methodology cross-references in TS 6.6.5.b. The NRC staff concludes that this is acceptable.

#### 3.2 Deletion of TS 6.6.5.b, Method 6

In its application dated May 15, 2009, the licensee proposed to delete TS 6.6.5.b, Method 6 which currently states:

- 6) "CESEC-Digital Simulation of a Combustion Engineering Nuclear Steam Supply System" (Methodology for Specifications 3.1.1.1 and 3.1.1.2 for Shutdown Margin, 3.1.1.4 for MTC, 3.1.3.1 for CEA Position, 3.1.3.6 for Regulating CEA and Group P Insertion Limits, and 3.2.4.b for DNBR Margin).

The guidance in NRC GL 88-16, in the section titled "DISCUSSION," states,

[A]n alternative to including the values of these cycle-specific parameters in individual specifications is provided ...

[T]his alternative consists of three separate actions to modify the plant's TS:  
(1) the addition of the definition of a named formal report that includes the values of cycle-specific parameter limits that have been established using an NRC-approved methodology and consistent with all applicable limits of the safety analysis ...

Based on the above discussion, the licensee stated in its letter dated May 15, 2009, that the CESEC methodology will be deleted from the ANO-2 TS 6.6.5.b, Method 6 listing, since it is not used to develop any of the ANO-2 safety analysis limits. The NRC staff concludes that this change is administrative in nature, and has no impact on the methods used to develop core design limits contained in the COLR and, therefore, is acceptable.

#### 3.3 Deletion of TS 6.6.5.b, Method 15

In its application dated May 15, 2009, the licensee proposed to delete TS 6.6.5.b, Method 15, which currently states:

- 15) "Calculative Methods for the CE Nuclear Power Large Break LOCA Evaluation Model – Improvement to 1999 Large Break LOCA EM Steam Cooling Model for Less Than 1 in/sec Core Reflood," CENPD-132,

Supplement 4-P-A, Addendum 1-P-A (Methodology for Specification 3.1.1.4 for MTC, 3.2.1 for Linear Heat Rate, 3.2.3 for Azimuthal Power Tilt, and 3.2.7 for ASI).

In its letter dated May 15, 2009, the licensee stated that the above reference is already listed as Method 4 in the TS. The NRC staff concludes that the proposed deletion of TS 6.6.5.b, Method 15, is administrative in nature and acceptable because this method is already listed as Method 4.

### 3.4 Correction of TS 6.6.5.b CENTS Reference to Equivalent Westinghouse Title

In its application dated May 15, 2009, the licensee proposed to revise TS 6.6.5.b, Method 7, which currently states:

- 7) "Technical Manual for the CENTS Code," CENPD 282-P-A (Methodology for Specifications 3.1.1.1 and 3.1.1.2 for Shutdown Margin, 3.1.1.4 for MTC, 3.1.3.1 for CEA Position, 3.1.3.6 for Regulating and Group P Insertion Limits, and 3.2.4.b for DNBR Margin).

The licensee proposed to revise TS 6.6.5.b, Method 7, to refer to the equivalent Westinghouse title for this methodology and, based on the deletion of TS 6.6.5.b, Method 6, as discussed in Section 3.2 above, Method 7 will be renumbered as Method 6. In Reference 3, Westinghouse changed the numbering of the topical report from a Combustion Engineering numbering scheme (CENPD 282-P-A) to a Westinghouse numbering scheme (WCAP-15996-P-A, Rev. 1) when the CENTS methodology was revised. In addition, the licensee proposes an editorial change to correct the omission of a closing parenthesis ")" in TS 6.6.5.b, Method 6. Based on the above, the revised method will state:

- 6) "Technical Manual for the CENTS Code," WCAP-15996-P-A, Rev. 1 (Methodology for Specifications 3.1.1.1 and 3.1.1.2 for Shutdown Margin, 3.1.1.4 for MTC, 3.1.3.1 for CEA Position, 3.1.3.6 for Regulating and Group P Insertion Limits, and 3.2.4.b for DNBR Margin).

The NRC staff reviewed the proposed change to TS 6.6.5.b, Method 7, and concludes it is acceptable because the proposed change is administrative in nature. In addition, the renumbering of the methods, in sequence, in TS 6.6.5.b, as well as the editorial change to restore missing punctuation, is acceptable because the changes are administrative in nature. This change does not have any material impact on the regulatory or technical meaning of the TS. Based on the above, the staff concludes that the proposed editorial change is acceptable because the requirements of 10 CFR 50.36 continue to be met.

In summary, a COLR TS amendment request should be submitted in accordance with the guidance specified in the GL 88-16. The COLR is a part of TS Administrative Controls which includes: (1) TS 6.6.5.a, parameters for core operating limits; (2) TS 6.6.5.b, NRC-approved analytical methods with approved date or revision to determine the core operating limits listed in TS 6.6.5.a; and (3) TS 6.6.5.d., the COLR, which shall be provided upon issuance for each reload cycle to the NRC. Therefore, the NRC staff requires the licensee, in its future COLR license amendment requests, to clearly identify and qualify those approved methodologies listed

in TS 6.6.5.b, which are used to calculate core operating limits listed in TS 6.6.5.a as described in Section 3.1 of this safety evaluation in terms of parenthetical COLR parameter-to-methodology cross-references.

### 3.5 Conclusion

Based on the above, the NRC staff concludes that the proposed changes in the license's license amendment request, as supplemented, are acceptable because the changes are administrative in nature and comply with NRC GL 88-16 guidance. In addition, the staff expects future COLR TS license amendment requests that the licensee clearly identify and qualify those approved methodologies listed in TS 6.6.5.b are directly used to calculate core operating limits listed in TS 6.6.5.a.

### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Arkansas State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding published in the *Federal Register* on June 30, 2009 (74 FR 31321). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

### 7.0 REFERENCES

1. Walsh, K. T., Entergy Operations, Inc., letter to U.S. Nuclear Regulatory Commission, "License Amendment Request to Revise Technical Specification 6.6.5, Core Operating Limits Report, Arkansas Nuclear One, Unit 2, Docket No. 50-368, License No. NPF-6," dated May 15, 2009 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML091350488).

2. Walsh, K. T., Entergy Operations, Inc., letter to U.S. Nuclear Regulatory Commission, "Response to Request for Additional Information Technical Specification 6.6.5, Core Operating Limits Report, Arkansas Nuclear One, Unit 2, Docket No. 50-368, License No. NPF-6," dated November 10, 2009 (ADAMS Accession No. ML093160202).
3. Walsh, K. T., Entergy Operations, Inc., letter to U.S. Nuclear Regulatory Commission, "Response to Request for Additional Information Associated With Core Operating Limits Report References, Arkansas Nuclear One Unit 2, Docket No. 50-368, License No. NPF-6, [and] Waterford Steam Electric Station Unit 3, Docket No. 50-382, License No. NPF-38," May 17, 2010 (ADAMS Accession No. ML101400028).
4. James, D. E., Entergy Operations, Inc., letter to U.S. Nuclear Regulatory Commission, "Response to Request for Additional Information License Amendment Request to Revise Technical Specification 6.6.5, Core Operating Limits Report, Arkansas Nuclear One, Unit 2, Docket No. 50-368, License No. NPF-6," dated March 11, 2008 (ADAMS Accession No. ML080710408).

Principal Contributor: Tai Huang

Date: May 25, 2010

May 25, 2010

Vice President, Operations  
Arkansas Nuclear One  
Entergy Operations, Inc.  
1448 S.R. 333  
Russellville, AR 72802

SUBJECT: ARKANSAS NUCLEAR ONE, UNIT NO. 2 - ISSUANCE OF AMENDMENT  
NO. 290 RE: LICENSE AMENDMENT REQUEST TO REVISE TECHNICAL  
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NO. ME1329)

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A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly *Federal Register* notice.

Sincerely,  
/RA/

N. Kaly Kalyanam, Project Manager  
Plant Licensing Branch IV  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-368

Enclosures:

1. Amendment No. 290 to NPF-6
2. Safety Evaluation

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