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March 13, 1980
IPN-80-30

Director of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. Albert Schwencer, Chief
Operating Reactors Branch No. 1
Division of Operating Reactors

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
I. E. Bulletin 79-07

Dear Sir:

In response to Mr. L. Olshan's verbal request on March 12, 1980 the following information regarding I. E. Bulletin 79-07 is provided. A detailed final report will be provided by April 11, 1980 as stated in our March 7, 1980 letter entitled, "Additional Information Needed on Pending Licensing Actions".

- 1) The reanalysis required by I. E. Bulletin 79-07 has been completed and indicates that 238 supports required modification. Approximately 15 of the 238 supports remain to be modified, all of which are accessible and have safety factors greater than 2 but less than 4. Modification of the remaining 15 supports will be completed by June 29, 1980.
- 2) The 07 reanalysis was performed such that the criteria of I. E. Bulletin 79-02 was also met.
- 3) As stated in our June 1, 1979 letter (IPN-79-30) four Velan valves were originally analyzed with weights which were significantly smaller than their actual weights. The corrected weights of these valves were included in the 07 reanalysis.
- 4) The UE&C ADLPIPE II Code was utilized to perform the 07 reanalysis.
- 5) "As-built" information was utilized in the 07 reanalysis.

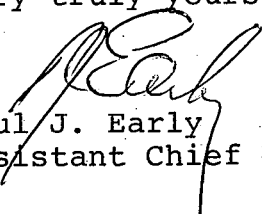
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- 6) The 07 reanalysis has has no effect on high energy line pipe break criteria since the latter is not based on pipe stress levels.

The Authority concludes that the results of the above re-evaluation and modifications to date provide adequate justification and assurance that the plant may continue to operate without undue risk to the public health and safety.

Very truly yours,


Paul J. Early
Assistant Chief Engineer-Projects

cc: Mr. T. Rebelowski
Resident Inspector
U. S. Nuclear Regulatory Commission
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