

ATTACHMENT I
PROPOSED TECHNICAL SPECIFICATION
CHANGES
RELATED TO SAFETY REVIEW
COMMITTEE
(APPENDIX A)

POWER AUTHORITY OF THE STATE OF NEW YORK
INDIAN POINT 3 NUCLEAR POWER PLANT
DOCKET NO. 50-286
NOVEMBER 28, 1979

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RECORDS

6.5.1.8 The Plant Operating Review Committee shall maintain minutes of each meeting and copies shall be provided to the Chairman of the SRC and Manager-Nuclear Operations.

6.5.2 SAFETY REVIEW COMMITTEE (SRC)

FUNCTION

6.5.2.1 The SRC shall collectively have the competence required to review problems in the following areas:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical engineering
- h. Electrical engineering
- i. Administrative controls and quality assurance practices
- j. Environment
- k. Civil/Structural Engineering
- l. Other appropriate fields associated with the unique characteristics of a nuclear power plant.
- m. Licensing

MEMBERSHIP

6.5.2.2 The SRC shall be composed of the following voting members:

Chairman:	Principal Nuclear Engineer - Staff
Vice-Chairman:	Director - Quality Assurance
Member:	Principal Nuclear Engineer-Projects
Member:	Radiological Engineer
Member:	Manager - Nuclear Operations
Member:	Principal Electrical Engineer - Staff
Member:	Director of Environmental Programs
Member:	Principal Civil/Structural Engineer
Member:	Nuclear Engineer - Staff (Secretary of SRC)
Member:	Principal Inservice Inspection Engineer - Nuclear Operations
Member:	Licensing Supervisor
Member:	Senior Nuclear Engineer - Staff
Member:	Principal Mechanical Engineer - Staff

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of facility operation following initial fuel loading and at least once per six months, thereafter.

QUORUM

6.5.2.6 A quorum of SRC shall consist of the Chairman or Vice-Chairman and six members, including alternates. No more than minority of the quorum shall have a direct line responsibility for the operation of the plant.

REVIEW

6.5.2.7 The SRC shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications of this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. Events requiring 24 hours written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the Plant Operating Review Committee.

ATTACHMENT II
SAFETY EVALUATION
RELATED TO SRC MEMBERSHIP

POWER AUTHORITY OF THE STATE OF NEW YORK
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Section I - Description of Modification

The membership on the Safety Review Committee has been expanded to include the Licensing Supervisor, the Principal Inservice Inspection Engineer and the Senior Nuclear Engineer - Staff. The SRC function of reviewing the Fire Protection Plan has been deleted as being redundant for the SRC's function is to audit the report of the independent reviewer.

Section II - Purpose of Modification

The purpose of this modification is to increase the breadth of expertise represented on the SRC and to comply with the requirements of 10 CFR §50.59(c) to report all changes to the Technical Specification.

Section III - Impact of the Change

The expansion in the SRC membership and the deletion of a redundant requirement will not alter the conclusion reached in the FSAR and SER Accident Analysis.

Section IV - Implementation of the Modification

The modification as proposed will not impact the ALARA or Fire Protection Program at IP3.

Section V - Conclusions

The incorporation of this modification: a) will not increase the probability nor the consequences of an accident or malfunction of equipment important to safety as previously evaluated in the Safety Analysis Report; b) will not increase the possibility for an accident or malfunction of a different type than any evaluated previously in the Safety Analysis Report; and c) will not reduce the margin of safety as defined in the basis for any Technical Specification.

Section VI - References

- (a) IP3 FSAR
- (b) IP3 SER