

LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 **1** | N Y T P S | **(2)** | 0 0 - 0 0 0 0 0 0 - 0 0 | **(3)** | 4 | | | | | | | | **(4)** | | | | **(5)**

7 8 9 14 15 25 26 30 37 CAT 38

LICENSEE CODE LICENSE NUMBER LICENSE TYPE

CON'T

REPORT SOURCE 0 1 7 8 L 6 0 5 0 0 0 2 8 6 7 0 9 1 2 7 8 8 1 0 0 2 7 8 9
60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | While operating at 100 percent power, the control room operator
0 3 | noticed that Loop No. 1 T-average was decreasing and found this
0 4 | was caused by failure of the hot leg RTD TE-410A. The RTD was
0 5 | declared inoperable in accordance with the requirements of tech-
0 6 | nical specification table 3.5-2, and the reactor protection trips
0 7 | associated with this particular channel were placed in the tripped
0 8 | condition.

7 8		SYSTEM CODE I A (11)		CAUSE CODE E (12)		CAUSE SUBCODE A (13)		COMPONENT CODE I N S T R U (14)						COMP. SUBCODE E (15)		VALVE SUBCODE Z (16)	
0 9		9 10		11 12		12 13		13 18						19 20		20 21	
7 8																	
17		EVENT YEAR 7 8 (21 22)		SEQUENTIAL REPORT NO. 0 3 1 (24 26)		OCCURRENCE CODE 0 3 (28 29)		REPORT TYPE L (30)		REVISION NO. 0 (32)							
17		21 22		23 24		24 26		27 28		28 29		30 31		32 33		33 34	
ACTION TAKEN X (18)		FUTURE ACTION Z (19)		EFFECT ON PLANT Z (20)		SHUTDOWN METHOD Z (21)		HOURS 0 0 0 0 (22 40)		ATTACHMENT SUBMITTED Y (23)		NPRD-4 FORM SUB. N (24)		PRIME COMP. SUPPLIER N (25)		COMPONENT MANUFACTURER S 2 6 0 (26 47)	
33 34		35 36		36 37		37 40		41 42		42 43		43 44		44 47		47 48	
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)																	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

1 0 Entry to containment was made and it was found that the manifold

1 1 stop valve 561A was leaking borated water onto the cabling and

1 2 junction box of the RTD in question. The leak was diverted by

1 3 wrapping the valve with insulating cloth, and the RTD cable and

1 4 box was rinsed with demineralized water.

FACILITY STATUS		% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION				
1	5	E	(28)	100	(29) NA	A	(31)	Operator Observation	(32)

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 2 33 3 34 NA NA

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	(37)		(38)		NA

7		8		9		11		12		13	
						PERSONNEL INJURIES					
						NUMBER		DESCRIPTION			
1	2	3	4	5	6	7	8	9	10	11	12
		0	0	0			(40)			NA	

7		8		9		11		12	
TYPE		DESCRIPTION							
1	9	2	42	NA					

7	8	9	10										
PUBLICITY													
ISSUED				DESCRIPTION (45)									
2	0	N	(14)	NA									
				NRC USE ONLY									

NRC USE ONLY

~~010000~~
NAME OF PREPARED

Floyd W. Gumblo

PHONE: 914 739 8200 x 217

ATTACHMENT 1

Docket No. 50-286

The Power Authority of
the State of New York

LER 78-031/03L-0

The plant was operating at 100 percent power in the steady state.

On September 12, 1978, the operator noted that T-average for loop No. 1 was steadily decreasing. The RTD was then declared inoperable as per Technical Specification Table 3.5-2. and the Reactor Protection System Trips associated with this channel were placed in the tripped condition. Containment entry was made, and it was observed that manifold stop valve 561A (as referred to in LER 78-030/03L-0) was leaking borated water onto the junction box and cabling for RTD No. TE-410A (Sostman and Company, Part No. 11901B-2).

The resistance of the RTD to ground was determined to be 250,000 ohms. This magnitude of resistance is sufficient to affect RTD reading.

The leak was diverted by wrapping the valve with insulating cloth. The junction box and cabling for the RTD were rinsed with demineralized water and allowed to dry. By the next morning, the resistance to ground was infinite. The T-average channel was functioning properly and was subsequently returned to service.

Performance of the reactor was not affected by this incident. No similar events have been recorded to date.