



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, DC 20555 - 0001

May 4, 2010

MEMORANDUM TO: Sherry Meador, Technical Secretary
Advisory Committee on Reactor Safeguards

FROM: Cayetano Santos, Chief */RA/*
Reactor Safety Branch
Advisory Committee on Reactor Safeguards

SUBJECT: MINUTES OF THE 570th MEETING OF THE ADVISORY
COMMITTEE ON REACTOR SAFEGUARDS (ACRS),
MARCH 4-6, 2010

I certify that based on my review of the minutes from the 570th ACRS Full Committee meeting, and to the best of my knowledge and belief, I have observed no substantive errors or omissions in the record of this proceeding subject to the comments noted below.

OFFICE	ACRS	ACRS:RSB/Sunsi
NAME	SMeador	CSantos/sam
DATE	05/ 03 /10	05/ 30 /10

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Date Certified: 05/04/10

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- Appendix I – *Federal Register* Notice
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During its 570th meeting, March 4-6, 2010, the Advisory Committee on Reactor Safeguards (ACRS) discussed several matters and completed the following reports and memoranda:

REPORTS

Reports to Gregory B. Jaczko, Chairman, NRC, from Said Abdel-Khalik, Chairman, ACRS:

- Draft Final Revision 1 to Digital Instrumentation and Control Interim Staff Guidance - 07: "Digital Instrumentation and Control Systems in Safety Applications at Fuel Cycle Facilities," dated March 25, 2010
- Draft Final Revision 1 of Regulatory Guide 1.141, "Containment Isolation Provisions for Fluid Systems," dated March 25, 2010
- Draft Revision 2 to Regulatory Guide 4.11 (DG-4016), "Terrestrial Environmental Studies for Nuclear Power Plants," dated March 25, 2010
- Status of Staff Rulemaking Efforts for Depleted Uranium and Other Unique Waste Streams, dated March 18, 2010
- Draft Final Revision 1 of Regulatory Guide 1.62, "Manual Initiation of Protective Actions," dated March 29, 2010

MEMORANDA

Memoranda to R. W. Borchardt, Executive Director for Operations, NRC, from Edwin M. Hackett, Executive Director, ACRS:

- Final Interim Staff Guidance ESP/DC/COL-ISG-015, "Post-Combined License Commitments," dated March 11, 2010
- Withdrawal of Regulatory Guide 8.6, dated March 11, 2010
- Draft Final Regulatory Guides 1.11, 1.126, 1.28, 1.65, and 3.39, dated March 11, 2010
- Proposed Revisions to Regulatory Guides 1.152, 2.6, 4.20, 8.10, 8.19, 8.4, and DG-1216, dated March 11, 2010
- Proposed Revision 2 to Regulatory Guide 1.54, dated March 15, 2010

MINUTES OF THE 570th MEETING OF THE
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS

ROCKVILLE, MARYLAND

The 570th meeting of the Advisory Committee on Reactor Safeguards (ACRS) was held in Conference Room 2B3, Two White Flint North Building, Rockville, Maryland, on March 4-6, 2010. Notice of this meeting was published in the *Federal Register* on February 23, 2010 (72 FR 8154-8155) (Appendix I). The purpose of this meeting was to discuss and take appropriate action on the items listed in the meeting schedule and outline (Appendix II). The meeting was open to public attendance.

A transcript of selected portions of the meeting is available in the NRC's Public Document Room at One White Flint North, Room 1F-19, 11555 Rockville Pike, Rockville, Maryland. Copies of the transcript are available for purchase from Neal R. Gross and Co., Inc., 1323 Rhode Island Avenue, NW, Washington, DC 20005. Transcripts are also available at no cost to download from, or review on, the Internet at <http://www.nrc.gov/ACRS/ACNW>.

ATTENDEES

ACRS Members: Dr. Said Abdel-Khalik (Chairman), Dr. J. Sam Armijo (Vice-Chairman), Mr. John Stetkar (Member-at-Large), Dr. George E. Apostolakis, Dr. Sanjoy Banerjee, Dr. Dennis Bley, Mr. Charles Brown, Dr. Michael Corradini, Dr. Dana A. Powers, Mr. Harold Ray, Dr. Michael Ryan, Dr. William Shack, and Mr. John Sieber. For a list of other attendees see Appendix III.

I. Chairman's Report (Open)

[Note: Mr. Edwin Hackett was the Designated Federal Official for this portion of the meeting.]

Dr. Said Abdel-Khalik, Committee Chairman, convened the meeting at 8:30 a.m. In his opening remarks he announced that the meeting was being conducted in accordance with the provisions of the Federal Advisory Committee Act. He reviewed the agenda items for discussion and noted that no written comments or requests for time to make oral statements from members of the public had been received. Dr. Bonaca also noted that a transcript of the open portions of the meeting was being kept and speakers were requested to identify themselves and speak with clarity and volume.

HIGHLIGHTS OF KEY ISSUES

II. Draft Final Interim Staff Guidance (ISG) on Fuel Cycle (ISG-07)

[Note: Mrs. Christina Antonescu was the Designated Federal Official for this portion of the meeting.]

The Committee met with representatives of the NRC staff to discuss Draft Final Revision 1 to ISG-07, "Digital Instrumentation and Control (DI&C) Systems in Safety Applications at Fuel Cycle Facilities," the changes made to this Guide since the August 21, 2009, DI&C Subcommittee meeting, and the resolution of public comments. Guidance for review of licensing applications of fuel cycle facilities consistent with the risk-informed licensing framework set forth in 10 CFR Part 70 is contained in NUREG 1520, "Standard Review Plan for the Review of a License Application for a Fuel Cycle Facility." Neither of these documents contains either analog or DI&C design criteria using industry codes and standards. Therefore, ISG-07 was developed in response to industry and NRC concerns regarding the need for consistency of review of fuel cycle facility applications.

ISG-07 provides guidance for reviewing the management measures for Items Relied on for Safety (IROFS) that use DI&C technology. Specifically, ISG-07 provides guidance on the following: (1) the acceptability of management measures regarding how "Cyber Security" is addressed for the protection of digital safety controls; (2) how redundant controls should be maintained "Independent" to prevent criticality events and other hazards; (3) how "Digital Communications" among safety controls should be isolated or protected; and (4) how the design of "High Quality Software" for DI&C safety applications should be ensured.

The Committee issued a report to the NRC Chairman, dated March 25, 2010, recommending that Revision 1 to DI&C ISG-07 not be issued until it is revised to state that any reduction in the level of rigorous management measures applied to redundant IROFS, relative to sole IROFS with the same design requirements, should be justified by a comprehensive analysis. The Committee also recommended that future efforts include the development of a systematic approach for identifying dependencies and common cause failures in IROFS and the development of an approach for structuring the individual scenario results of an Integrated Safety Assessment (ISA) to facilitate review and understanding of the associated risk significance.

III. Draft Final Regulatory Guide (RG) 1.141, "Containment Isolation Provisions for Fluid Systems"

[Note: Ms. Zena Abdullahi was the Designated Federal Official for this portion of the meeting.]

The Committee met with representatives of the NRC staff to discuss Draft Final Revision 1 of Regulatory Guide (RG) 1.141, "Containment Isolation Provisions for Fluid Systems." This Guide provides updated guidance on acceptable design, testing, and maintenance requirements for the isolation of fluid systems that penetrate the primary containment. The staff described the specific regulatory positions in the RG as well as the associated industry guidance. The

proposed changes to RG 1.141 would allow relief valves to serve as containment isolation valves in the forward direction, provided that the lift setpoint is at least 50 percent greater than the containment design pressure. The use of relief valves in the forward flow direction to perform a containment isolation function poses the risk of creating pathways for bypassing containment.

The Committee issued a report to the NRC Chairman on this matter dated March 25, 2010, recommending that Revision 1 of RG 1.141 be issued after it is revised to include additional provisions similar to those in the 1989 edition of ANSI/ANS 56.2, Section 4.7.5, "Relief Valves in the Forward Flow Direction." The Committee also recommended that appropriate portions of NUREG-0800 be revised consistent with this recommendation.

IV. Draft Revision 1 to Regulatory Guide 4.11, "Terrestrial Environmental Studies for Nuclear Power Stations"

[Note: Mr. Derek Widmayer was the Designated Official for this portion of the meeting.]

The Committee met with representatives of the NRC staff to discuss Draft Revision 2 to RG 4.11 [Draft Guide (DG) - 4016], "Terrestrial Environmental Studies for Nuclear Power Plants." The staff described their basis for revising RG 4.11 and its relationship to other NRC environmental guidance. DG-4016 updates guidance for the conduct of terrestrial environmental studies to support analyses presented in a licensee's environmental report in support of siting new nuclear power plants. DG-4016 addresses siting support, baseline investigations, identification of important species and habitats, impact analyses, monitoring, and decommissioning. The Committee was interested in the guidance because no revisions of the guide have been issued since 1977. The Committee learned that there is no guidance document that addresses aquatic environmental studies.

The Committee issued a report to the NRC Chairman on this matter dated March 25, 2010, recommending that the Guide be issued for public comment after certain revisions are made. The Committee also recommended that the staff develop a complementary guide on aquatic environmental studies.

V. Status of Rulemaking for Disposal of Depleted Uranium and Other Unique Waste Streams

[Note: Mr. Neil Coleman was the Designated Federal Official for this portion of the meeting.]

The Committee met with representatives of the NRC staff to discuss the status of rulemaking for Depleted Uranium (DU) and other unique waste streams. In 2005 the Commission directed the staff to consider whether the quantities of DU in the waste streams from uranium enrichment facilities warrant amending 10 CFR Part 61.55(a)(6) or the waste classification tables of section 61.55(a). The staff conducted technical analyses for a variety of site characteristics and concluded that near-surface disposal of large quantities of DU can be appropriate in some cases, but cannot be done at all sites. The staff recommended a limited rulemaking to revise 10 CFR Part 61 to require a licensee or applicant to conduct site-specific analyses that address

the characteristics of the site and the proposed waste form prior to disposal of large quantities of DU. In September 2009, the staff conducted workshops in Bethesda, Maryland, and Salt Lake City, Utah, to inform the public about the rulemaking status and the issues regarding unique low-level waste streams, including DU. The staff plans to develop interim guidance for use until the rulemaking is complete and to offer public demonstrations of the models that support their efforts to date. The staff plans to respond to requests for technical assistance from Agreement States.

The Committee issued a report to the NRC Chairman on this matter dated March 18, 2010, recommending that the staff continue their efforts to risk-inform the regulations for disposal of depleted uranium based on site-specific, realistic performance assessments with appropriate consideration of uncertainties.

VI. Draft ACRS Report on the NRC Safety Research Program

[Note: Dr. Hossein Nourbakhsh was the Designated Federal Official for this portion of the meeting.]

The ACRS provides the Commission a biennial report presenting the Committee's observations and recommendations concerning the overall NRC Safety Research Program. During the March 2010 meeting, the Committee completed its biennial review and evaluation of the Reactor Safety Research Program sponsored by the NRC. The Committee will issue its 2010 biennial report to the Commission entitled, "Review and Evaluation of the Nuclear Regulatory Commission Safety Research Program," by April 15, 2010. The final report will be published as NUREG-1635, Vol. 9.

VII. Digital Instrumentation and Control (I&C) Design Acceptance Criteria (DAC) Inspection Methodology (Open)

[Note: Mrs. Christina Antonescu was the Designated Official for this portion of the meeting.]

The Committee met with representatives of the NRC staff to review the proposed DI&C Design Acceptance Criteria (DAC) inspection methodology and the plans for piloting this approach at the South Texas Project (STP). The DAC Working Group in NRO has developed a viable methodology for DAC review/inspection/resolution, and the staff plans to apply it to an actual DI&C DAC product at STP Units 3/4. The staff provided an overview of DI&C DAC inspection strategies which generally mirror the digital system/software development lifecycle found in the Standard Review Plan, Branch Technical Position 7-14 (Guidance for Review of Digital I&C Systems). The Inspection Strategy Documents borrow from industry standards, regulatory guidance and staff expertise, and feature descriptions, key attributes, and inspection techniques. These documents are intended to support lifecycle phase-oriented inspection and provide a technical basis for subsequent development of a new ITAAC Inspection Procedure (IP 65000 series) for I&C DAC. Following completion of the pilot effort, the DAC Working Group will identify and incorporate necessary enhancements into a generic methodology for future application.

This was an information briefing. No Committee action was necessary. The Committee plans to continue its review of DAC inspection guidance at a future meeting. The Committee has requested that, upon completion, copies of the Inspection Strategy Documents be provided by the staff for the Committee's review.

VIII. New Advanced Reactor Designs

[Note: Mrs. Kathy Weaver was the Designated Federal Official for this portion of the meeting.]

The Committee met with representatives of the NRC staff to discuss the Agency's Advanced Reactor Program. The reactor technologies addressed by the Advanced Reactor Program include high-temperature gas-cooled reactors (i.e. the Next Generation Nuclear Plant), integral pressurized water reactors (PWRs), and sodium-cooled fast reactors. The high-temperature gas-cooled reactors are associated with the Energy Policy Act of 2005 and the Next Generation Nuclear Plant (NGNP). The staff's current NGNP activities include evaluating existing requirements and guidance to identify needed changes, identifying significant policy and technical issues, and developing an overall licensing plan. The staff is also reviewing NGNP white papers associated with issues such as defense-in-depth, fuel design, high temperature materials, and analytical code verification and validation. Integral pressurized water reactors are PWRs with nuclear steam supply components (e.g. steam generator, reactor coolant pumps, etc.) housed within the reactor vessel. The staff is in pre-application discussions with vendors regarding the Westinghouse IRIS, NuScale, and B&W mPower designs. The NRC's activities associated with sodium fast reactors are limited, but the staff has had some pre-application interactions with vendors regarding the Toshiba 4S and General Electric PRISM designs.

The staff described some of the policy and technical issues associated with licensing these new reactor designs. The staff concluded their presentation by identifying topics for future ACRS interactions such as the high-temperature gas-cooled-reactor research plan and the resolution of specific policy or technical issues. This was an information briefing. No Committee action was necessary.

IX.. Meeting with the NRC Executive Director for Operations

[Note: Mr. Edwin Hackett was the Designated Federal Official for this portion of the meeting.]

The ACRS met with the EDO and Deputy EDOs to discuss items of mutual interest including the Agency's budget priorities, oversight and licensing of reactors, new fuel cycle facilities, nuclear materials users, spent fuel management, decommissioning, and enhancing public participation. This was an information briefing. No Committee action was necessary.

VI. Executive Session

[Note: Mr. Edwin Hackett was the Designated Federal Official for this portion of the meeting.]

A. Reconciliation of ACRS Comments and Recommendations/EDO Commitments

- The Committee considered the EDO's response of February 1, 2010, to comments and recommendations included in the December 10, 2009, ACRS report on the draft final Revision 1 of Regulatory Guide 1.151, "Instrument Sensing Lines." The Committee decided that it was satisfied with the EDO's response.
- The Committee considered the EDO's response of December 11, 2009, to comments and recommendations included in the November 13, 2009, ACRS report on the status of the ACRS review of the Westinghouse AP1000 Design Certification Amendment. The Committee decided that it was satisfied with the EDO's response.
- The Committee considered the EDO's response of January 13, 2010, to comments and recommendations included in the December 10, 2009, ACRS report on the safety aspects of the license renewal application for the Prairie Island Nuclear Generating Plant, Units 1 and 2. The Committee decided that it was satisfied with the EDO's response.
- The Committee considered the EDO's response of December 3, 2009, to comments and recommendations included in the October 22, 2009, ACRS report on the closure of Steam Generator Action Plan Items 3.1k, 3.4, 3.5, 3.10, 3.11, and 3.12 and staff closeout of the Steam Generator Action Plan. The Committee decided that it was satisfied with the EDO's response.

B. Report of the Planning and Procedures Subcommittee Meeting

Review of the Member Assignments and Priorities for ACRS Reports and Letters for the March ACRS Meeting

Member assignments and priorities for ACRS reports and letters for the March ACRS meeting were discussed. Reports and letters that would benefit from additional consideration at a future ACRS meeting were also discussed.

Anticipated Workload for ACRS Members

The anticipated workload for ACRS members through June 2010 were discussed and the objectives were to:

- Review the reasons for the scheduling of each activity and the expected work product and to make changes, as appropriate
- Manage the members' workload for these meetings
- Plan and schedule items for ACRS discussion of topical and emerging issues

Regulatory Guides and Interim Staff Guidance

a) Draft Final Regulatory Guide

The staff plans to issue the following Draft Final Regulatory Guides and would like to know whether the Committee wants to review these Guides prior to being issued final.

Draft Final Regulatory Guide 1.11, "Instrument Lines Penetrating the Primary Reactor Containment"

Regulatory Guide (RG) 1.11 has been revised to provide additional wording in Section B, Discussion, and Regulatory Position C.1. The additional wording clarifies that the requirements for redundancy, independence, and testability are for the system to which the instrument lines belonged and not to an instrument line itself. The instrument line design should support achievement of these design requirements for the systems they support.

Based on his review of this RG, Mr. Ray recommends that the Committee review the draft final revision to this Guide.

Draft Final Regulatory Guide 1.216, "Containment Structural Integrity Evaluation for Internal Pressure Loadings above Design-Basis Pressure"

RG 1.216 is a new guide that was issued as DG-1203 for public comments on December 9, 2008. The public comment period closed on February 9, 2009. Comments were received and RG 1.216 was revised accordingly. RG 1.216 describes methods that the NRC staff considers acceptable for (1) predicting the internal pressure capacity for containment structures above the design-basis accident pressure, (2) demonstrating containment structural integrity related to combustible gas control, and (3) demonstrating containment structural integrity to meet the Commission's performance goals related to the prevention and mitigation of severe accidents. RG 1.216 does not address requirements and guidance for the structural evaluation of containments for design-basis pressure. Since issuance of DG-1203, the scope has been revised to address only new light water reactor designs, Regulatory Position 3 has been changed, Regulatory Position 4 has been eliminated, the title has been changed, and other minor clarifications.

Based on his review of this RG, Dr. Shack recommends that the Committee review the draft final revision to this Guide.

Draft Final Regulatory Guide 1.126, "An Acceptable Model and Related Statistical Methods for the Analysis of Fuel Densification"

RG 1.126 describes an analytical model and related assumptions and procedures that the staff considers acceptable for predicting the effects of fuel densification in light-water-cooled nuclear power reactors. The guide describes statistical methods related to product sampling that will ensure that this and other approved analytical models will adequately describe the effects of densification for each initial core and reload fuel quantity produced.

Based on his review of this RG, Dr. Armijo recommends that the Committee not review the draft final revision to this Guide.

Draft Final Regulatory Guide 1.28, "Assurance Program Requirements (Design and Construction)"

RG 1.28 extends the scope of the NRC's endorsement to include NQA 1, Part II, which contains amplifying QA requirements for certain specific work activities (which had been found in NQA-2) that occur at various stages of a facility's life. The work activities include, but are not limited to, management, planning, site investigation, design, computer software use, commercial grade dedication, procurement, fabrication, installation, inspection, and testing. Regulatory Guide 1.28, Appendix A, "Evolution of Quality Assurance Standards and the Endorsing Regulatory Guides," gives an overview and continuation of the history and consolidation of NRC-endorsed QA standards.

Based on his review of this RG, Mr. Stetkar recommends that the Committee not review the draft final revision to this Guide.

Draft Final Revision 1 to Regulatory Guide 1.65 (DG-1211), "Materials and Inspections for Reactor Vessel Closure Studs"

RG 1.65 has been revised to include a position previously established for license renewal and provided in NUREG/CR-1801, "Generic Aging Lessons Learned." This position is that design conservatism should be exercised in determining the sizing of the studs so that the measured ultimate tensile strength does not reach a level that would make the studs susceptible to stress corrosion cracking. This revision also addresses the use of lubricants as reflected in NUREG-0800, "Standard Review Plan," Section 3.13, "Threaded Fasteners - ASME Code Classes 1, 2, and 3." The updated guidance incorporates operating experience regarding the types of lubricants that are acceptable and those that have been found to be detrimental. Finally, a number of references that were not cited in the original 1973 Guide have been added so make the guide easier to use.

Based on his review of this RG, Dr. Armijo recommends that the Committee not review this Guide.

Draft Final Regulatory Guide 3.39 (DG-3038), "Standard Format and Content of License Applications for Plutonium Processing and Fuel Fabrication Facilities"

RG 3.39 describes information acceptable to the NRC staff for review of a Safety Analysis Report (SAR) for mixed oxide fuel fabrication facilities. It also provides the standard format and content of an SAR and related documents that may be submitted as part of an application to construct or modify and operate a mixed oxide fuel fabrication facility.

Based on his review of this RG, Dr. Powers recommends that the Committee not review this Guide.

b) Draft Regulatory Guides

The staff plans to issue the following Draft Regulatory Guides for public comment and would like to know whether the Committee wants to review these Guides prior to being issued for public comment.

Proposed Revision 1 to RG 4.20 (DG-4018), "Personnel Constraint on Releases of Airborne Radioactive Materials to the Environment for Licensees Other than Power Reactors"

DG-4018 is a proposed Revision 1 to RG 4.20, which was issued in 1996. New tools have been developed since the issuance of RG 4.20. Several important changes are being proposed. The method for demonstrating compliance based on a simple ratio to the concentrations in 10 CFR Part 20 Appendix B has been revised from a ratio of less than or equal to 0.2 to be a ratio of less than or equal to 0.1. This is because the concentrations in 10 CFR Part 20, Appendix B for effluents were derived for a 100 mrem exposure and includes a factor of 2 for consideration of sensitive populations. The introductory text to Appendix B states the concentration results in 50 mrem, which does not take into consideration the factor of 2 included in the derivation. This change will allow demonstration that the 10 mrem/yr constraint is met for sensitive populations which is considered consistent with 10 CFR Part 20. The proposed revision has an enhanced discussion of the computer models, allows use of guidance other than ICRP 30, and allows use of site specific meteorological data and physical features. All references were reviewed and revised, as appropriate. The impact of these changes is to make the guidance more consistent with the regulation and incorporate appropriate current industry practice.

Based on his review of this Proposed Regulatory Guide, Dr. Ryan recommends that the Committee review the proposed revision to this Guide before being issued for public comment.

Proposed Revision 2 to RG 8.19 (DG-8034), "Occupational Radiation Dose Assessment in Light-Water Reactor Power Plants Design Stage Person-Sievert (Man-Rem) Estimates"

DG-8034 is a proposed Revision 2 of RG 8.19, which was issued in 1979. This Guide describes a method acceptable to the NRC staff for assessing the occupational radiation dose as part of the ongoing design review process involved in designing a light-water reactor (LWR) to ensure that occupational radiation exposures are as low as reasonably achievable (ALARA). This proposed revision updates the dose assessment based on design changes, e.g., shielding design and layout of equipment. References to 10 CFR 52.47, "Contents of Applications; Technical Information," and Part I, "Standard Format and Content of Combined License Applications," of Regulatory Guide 1.206, "Combined License Applications for Nuclear Power Plants," for certified designs have been added to provide more detail as to the information the NRC staff needs to evaluate applications. The terminology has been revised to be consistent with RG 1.206.

Based on his review of this Proposed RG, Dr. Ryan recommends that the Committee review the proposed revision to this Guide before being issued for public comment.

Proposed Revision 3 to RG 1.152 (DG-1249), "Criteria for Use of Computers in Safety Systems of Nuclear Power Plants"

DG-1249 is a proposed Revision 3 of RG 1.152 which was issued as Revision 2 in January 2006. DG-1249 acknowledges that 10 CFR 73.54, "Protection of Digital Computer and Communication Systems and Networks," requires licensees to develop cyber-security plans and programs to protect critical digital assets, including digital safety systems, from malicious cyber attacks. RG 5.71, "Cyber Security Programs for Nuclear Facilities," provides guidance to meet the requirements of 10 CFR 73.54. The combination of DG-1249 and the programmatic provisions under 10 CFR 73.54 should address the secure design, development, and operation

of digital safety systems. To address these issues, DG-1249 eliminates all reference to cyber security, malicious activity, or attacks to remove any duplication between the documents. DG 1249 emphasizes RG 1.152's focus on security for the protection of digital safety systems against non-malicious events. DG-1249 deletes Regulatory Positions 2.6 through 2.9, which address security in the operational phases of a system's life cycle. Licensing is complete once the Factory Acceptance Testing is concluded. The licensee's cyber security programs should now address these considerations to meet the requirements of 10 CFR 73.54.

Based on his review of this Proposed RG, Mr. Brown recommends that the Committee review the proposed revision to this Guide after reconciliation of public comments.

Proposed Revision 1 to RG 2.6 (DG-2004), "Emergency Planning for Research and Test Reactors"

ANSI/ANS 15.16-2008, as revised and endorsed by this RG, includes a few key changes and experience accumulated since the issuance of the earlier version (ANSI/ANS 15.16-1982). Primarily, the changes in the standard and the RG are that they are now consistent with the changed dose limits of 10 CFR Part 20, "Standards for Protection against Radiation," and with enhanced physical security practices for reactor facilities. Additionally, NRC Information Notice 92-79, "Non-Power Reactor Emergency Event Response," describes an event that required interface with the public and highlights the need for licensees to quickly apprise the NRC of the circumstances related to declaring an emergency classification.

Based on his review of this Proposed Regulatory Guide, Mr. Stetkar recommends that the Committee review the proposed revision to this Guide after reconciliation of public comments.

Proposed Revision 1 to RG 8.4 (DG-8036), "Personnel Monitoring Device - Direct-Reading Pocket Dosimeters"

DG-8036 is a proposed Revision 1 to RG 8.4 which was issued in 1973. It updates the ANSI Standard N322-2009 instead of the standard referenced in the previous version (ANSI N13.5-1972). The proposed revision provides guidance on direct-reading pocket dosimeters and excludes the indirect-reading pocket dosimeters which were included in the previous version of this guide. The indirect-reading pocket dosimeters are essentially no longer used. The proposed revision includes guidance for test and calibration frequency, limits for accuracy testing, training of users, and record keeping of the dosimeters testing results.

Based on his review of this Proposed RG, Dr. Ryan recommends that the Committee review the proposed revision to this Guide after reconciliation of public comments.

Proposed Revision 2 to RG 8.10 (DG-8033), "Philosophy for Maintaining Occupational Radiation Exposures As Low As Is Reasonably Achievable"

DG-8033 is a proposed Revision 2 to RG 8.10. It updates the regulation citation and the supporting ALARA Regulatory Guides, adds a discussion on the ALARA concept, includes worker responsibilities, and emphasizes management commitment to ALARA with the initial design of the facility and its operating procedures. This includes describing management responsibilities related to resources, independent audits, audit recommendations, and training

that demonstrate their commitment. Additional processes performed by the Radiation Protection Manager/Radiation Safety Officer and the Radiation Protection staff were added that strengthen their responsibility to maintain ALARA. These processes relate to work plans, procedures, investigation, documentation and exploring, and incorporating best practices.

Based on his review of this Proposed Regulatory Guide, Dr. Ryan recommends that the Committee review the proposed revision to this Guide after reconciliation of public comments.

Proposed DG-1216 (New RG), "Plant-Specific Applicability of Transition Break Size Specified in 10 CFR 50.46a"

DG-1216 is proposed as a new Regulatory Guide. Amendments to 10 CFR 50 are being proposed to establish procedures and criteria for requesting changes in plant design and procedures based on results of the new analyses of emergency core cooling system (ECCS) performance. This proposed RG describes a method that the staff considers acceptable for demonstrating that the generic transition break size specified in 10 CFR 50.46a is applicable to a specific plant and provides for licensees to implement a voluntary, risk-informed alternative to the requirements for analyzing the performance of the ECCS during loss-of-coolant accidents. The detailed guidance provided in this proposed guide will allow licensees to more easily evaluate the benefits of using the provisions in the proposed 10 CFR 50.46(a) to make plant changes, while allowing the agency to minimize its review efforts of applications submitted for licensees wishing to use the alternative requirements of the rule.

Based on his review of this Proposed RG, Dr. Shack recommends that the Committee review the proposed revision to this Guide after reconciliation of public comments.

Proposed Revision 2 to RG 1.54 (DG-1242), "Service Level I, II, and III Protective Coatings Applied to Nuclear Power Plants"

DG-1242 is a proposed Revision 2 to RG 1.54. This guide was first issued in June 1973. Revision 1 of RG 1.54 was issued in July 2000 to provide endorsement and regulatory positions on the American Society for Testing and Materials (ASTM) standards as applicable to nuclear power plant (NPP) protective coatings. Since the issuance of RG 1.54, Revision 1, many of the ASTM standards endorsed by RG 1.54 have been updated to reflect the current industry practice. This revision is being developed to provide NRC regulatory positions on the updated ASTM standards for the selection, qualification, application, and maintenance of protective coatings in NPPs.

Based on his review of this Proposed Regulatory Guide, Dr. Armijo recommends that the Committee not review the proposed draft and final revisions to this Guide.

c) Withdrawal of Regulatory Guides

The staff plans to withdraw the following Regulatory Guide and would like to know whether the Committee wants to review this Guide prior to being withdrawn.

Regulatory Guide 8.6. "Standard Test Procedure for Geiger-Müller Counters"

RG 8.6 was issued July 1973 and endorsed the test procedures specified in the American Nuclear Standards Institute (ANSI) N42.3-1969, "Test Procedure for Geiger-Müller Counters." In the 1970s, the Geiger-Müller (GM) counters were the main radiation detection instruments used by licensees. At that time, radiation protection programs needed the guidance included in RG 8.6 because there was limited information available on the use and maintenance of these counters. However, since the 1970s, technology has changed radically, and currently, in addition to GM counters, there are many types of radiation detection and measurement instruments used. Most of them are fairly complex to operate, maintain, and calibrate. Generally, the NRC does not provide specific guidance for the technical testing or calibration of radiation detection and measurement equipment. Any such guidance would soon become outdated, since the development of these instruments is continuously advancing, producing new models tailored to a whole range of specialized clienteles. As is evidence of this, the ANSI N42.3-1969 has not been revised and therefore it is outdated and no longer useful. The manufacturers provide instructions and training for testing and calibration of each new instrument. Also, since the 1970s, the industry has gained extensive experience in the characteristics of the GM counters and other instruments. In addition, organizations such as the National Institute of Standards and Technology and other private groups offer calibration services to those who lack in-house training and experience in testing and calibrating instruments.

Based on his review of the staff's basis for the proposed withdrawal of this Guide, Dr. Ryan recommends that the Committee not object to the staff's proposal to withdraw this Guide.

d) Interim Staff Guidance

The staff issued the following Interim Staff Guidance (ISG) as final and would like to know whether the Committee wants to review this Guidance.

ISG-015, "Post-Combined License Commitments"

The purpose of this ISG is supplement the guidance provided to the NRC staff in Section 1.0, "Introduction and Interfaces," of NUREG-0800, "Standard review Plan for the Review of safety Analysis Reports for Nuclear Power Plants," concerning the review of applications to support early site permit (ESP), design certification (DC) and combined license (COL) applications. In addition, this ISG supplements the guidance provided in Section C.III.4 of regulatory Guide (RG) 1.206, "Regulatory Guide for Combined License Applications for Nuclear Power Plants," June 2007.

Based on his review of ISG-015, Dr. Bley recommends the Committee to review this ISG.

Staff Requirements Memorandum

Attached is the Staff Requirements Memorandum (SRM-M091204) resulting from the ACRS meeting with the Commission on December 4, 2009. In this SRM, the Commission stated that "there were no requirements identified for staff action."

ACRS Meeting With the Commission

The ACRS is scheduled to meet with the Commission between 1:30 and 3:30 p.m., on Thursday, June 9, 2010 to discuss items of mutual interest. A list of topics proposed by the ACRS staff is provided below:

Overview (Abdel-Khalik)

- Accomplishments
- New Reactors Review Activities
- License Renewal/Power Uprates
- Ongoing/Future Activities

Risk-Informed Performance-Based Fire Protection (RG 2.105) (Stetkar)
Safety Research Program (Powers)
BWROG COP Methodology (Shack)
Status of Rulemaking for Disposal of Depleted Uranium (Ryan)

Changes to the Bylaws Regarding Election of Vice Chairman

According to the current bylaws, Chairman and Vice Chairman “shall be elected by a numerical majority of the current membership using a secret ballot.” For Member-at-Large “nominations will be made from the floor and seconded by the Committee Members. Subsequent to the nominations, the Member-at-large shall be elected by a numerical majority of the members attending the meeting using a secret ballot.”

During the December 2009 meeting, the Members discussed possible changes to the bylaws such as requiring nominations prior to the election of the Vice Chairman.

Changes to the bylaws may be proposed by any member. A proposed amendment to the bylaws should be distributed to the members by the Executive Director and scheduled for discussion at the next full Committee meeting. The final proposed amendment is to be voted on not earlier than the first regular meeting after it has been presented to the full Committee. A vote of two-thirds of the current membership is required to approve an amendment.

NRR Regulatory Information Conference

The 22nd Annual Regulatory Information Conference (RIC) will be held at the Bethesda North Marriott on March 9-11, 2010. A draft schedule of the technical sessions is attached (p. 22). The ACRS-sponsored session is scheduled for Thursday, March 11, from 11:00 a.m. to 1:00 p.m. The agenda for this session:

RIC SESSION TITLE: **ACRS: The Latest Chapter**

- (1) Dr. Said Abdel-Khalik, ACRS Chairman
Presentation Title - ACRS: Who We Are and What We Do
- (2) Frank Akstulewicz, Deputy Division Director, NRC Office of New Reactors
Presentation Title- ACRS Contributions to the New Reactor Licensing Process

- (3) Mark Manoleras, Director, Beaver Valley Site Engineering
Presentation Title- ACRS Interface: Utility Perspective
- (4) Dr. Edwin Lyman, Senior Staff Scientist, Union of Concerned Scientists
Presentation Title- The Role of the ACRS: A Public Interest View

Status of Solicitation for New Members

The ACRS has received permission from the Commission to extend the solicitation for a new member and to solicit for multiple positions. The new solicitation was published in the Federal Register on January 13, 2010, and will close after 90 days.

ACRS Feedback Form

The ACRS Chair is proposing a way for members to provide feedback on how meetings were supported by the technical and administrative staff.

Length and Content of Subcommittee Minutes

Member Dennis Bley is proposing a more straight forward approach for the preparation and certification of subcommittee meeting minutes. At issue is the length of some of the subcommittee minutes currently being generated, the time it takes the staff to prepare such documents, and the overall value added by these documents. After consulting the ACRS bylaws, he is of the opinion that it would seem reasonable for the subcommittee minutes to include a short "SUMMARY OF THE MEETING" documenting who presented what topics and a summary of the most meaningful comments of members. As long as the transcript and presentations are attached, there should be no need for the lengthy minutes to be prepared.

Availability of Committee's Draft Materials Prior to Meetings

Member Dennis Bley has raised the issue of availability and handling of ACRS draft materials.

Under FACA, Committee materials (including drafts) must be made available to the public when requested. However, FACA also provides for "preparatory work" to be performed in a non-public manner.

Annual Visit to a Nuclear Plant and Meeting with the Regional Administrator

Each year, the members visit a nuclear power plant site and meet with the Regional Administrator to discuss items of mutual interest. In 2010, the members plan to visit a plant in Region I.

Proactive Initiatives

Traditionally, Senior Technical Fellows performed special technical reviews for ACRS. In 2002, in order to meet the agency's A-76 goals, the position was outsourced and since then a technical contractor has been available to provide professional engineering and scientific work in support of the ACRS activities. The Committee may assign specific technical work to this contract resource.

C. Future Meeting Agenda

Appendix IV summarizes the proposed items endorsed by the Committee for the 571st ACRS Meeting, April 8-10, 2010.

A list of documents that were provided to the Committee during the 570th ACRS Meeting is listed in Appendix V.

The meeting was adjourned at 8:00 p.m. on Friday, March 5, 2010.

Rockville, Maryland 20852. Publicly available records will be accessible electronically from the Agencywide Documents Access and Management System (ADAMS) Public Electronic Reading Room on the Internet at the NRC Web site, <http://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC PDR Reference staff by telephone at 1-800-397-4209 or 301-415-4737, or send an e-mail to pdr.resource@nrc.gov.

Dated at Rockville, Maryland, this 16th day of February 2010.

For the Nuclear Regulatory Commission.

Carl F. Lyon,

*Project Manager, Plant Licensing Branch IV,
Division of Operating Reactor Licensing,
Office of Nuclear Reactor Regulation.*

[FR Doc. 2010-3497 Filed 2-22-10; 8:45 am]

BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

Advisory Committee on Reactor Safeguards

In accordance with the purposes of Sections 29 and 182b of the Atomic Energy Act (42 U.S.C. 2039, 2232b), the Advisory Committee on Reactor Safeguards (ACRS) will hold a meeting on March 4-6, 2010, 11545 Rockville Pike, Rockville, Maryland. The date of this meeting was previously published in the **Federal Register** on Monday, October 14, 2009, (74 FR 52829-52830).

Thursday, March 4, 2010, Conference Room T2-B1, Two White Flint North, Rockville, Maryland

8:30 a.m.–8:35 a.m.: *Opening Remarks by the ACRS Chairman* (Open)—The ACRS Chairman will make opening remarks regarding the conduct of the meeting.

8:35 a.m.–10 a.m.: *Draft Final Interim Staff Guidance (ISG) on Fuel Cycle (ISG-7)* (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff and the Nuclear Energy Institute (NEI) regarding draft final ISG on Fuel Cycle, NRC staff's resolution of public comments, and related matters.

10:15 a.m.–12 p.m.: *Draft Final Regulatory Guide (RG) 1.141, "Containment Isolation Provisions for Fluid Systems"* (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding draft final RG 1.141,

"Containment Isolation Provisions for Fluid Systems," NRC staff's resolution of public comments, and related matters.

1 p.m.–2 p.m.: *Draft Final Revision 1 to Regulatory Guide 4.11, "Terrestrial Environmental Studies for Nuclear Power Stations"* (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding draft Revision 1 to RG 4.11, "Terrestrial Environmental Studies for Nuclear Power Stations," NRC staff's resolution of public comments, and related matters.

2 p.m.–3:15 p.m.: *"Status of Rulemaking for Disposal of Depleted Uranium and Other Unique Waste Streams"* (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding the status of rulemaking efforts addressing disposal of depleted uranium and other unique waste streams, and related matters.

3:30 p.m.–5 p.m.: *Draft ACRS Report on the NRC Safety Research Program* (Open)—The Committee will discuss the draft ACRS Report on Safety Research Program.

5 p.m.–7 p.m.: *Preparation of ACRS Reports* (Open)—The Committee will discuss proposed ACRS reports on matters discussed during this and the previous meeting (February 2010).

Friday, March 5, 2010, Conference Room T2-B1, Two White Flint North, Rockville, Maryland

8:30 a.m.–8:35 a.m.: *Opening Remarks by the ACRS Chairman* (Open)—The ACRS Chairman will make opening remarks regarding the conduct of the meeting.

8:35 a.m.–10:15 a.m.: *Digital Instrumentation and Control (I&C) Design Acceptance Criteria (DAC) Inspection Methodology* (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding Digital I&C DAC Inspection Methodology.

10:30 a.m.–12 p.m.: *New Advanced Reactor Designs* (Open)—The Committee will hear presentations by and hold discussions with representatives of the NRC staff regarding new advanced reactor designs such as NuScale, Iris, Babcock and Wilcox Modular, Hyperion, Toshiba's 4S, and General Electric's Prism.

1:30 p.m.–3 p.m.: *Meeting with the NRC Executive Director for Operations*

(Open)—The Committee will meet with the NRC Executive Director for Operations (EDO) and Deputy EDOs to discuss topics of mutual interest.

3 p.m.–4:30 p.m.: *Future ACRS Activities/Report of the Planning and Procedures Subcommittee* (Open/Closed)—The Committee will discuss the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the Full Committee during future ACRS meetings, including anticipated workload and member assignments, and related matters. [**Note:** A portion of this session may be closed pursuant to 5 U.S.C. 552b (c)(2) and (6) to discuss organizational and personnel matters that relate solely to internal personnel rules and practices of ACRS, and information the release of which would constitute a clearly unwarranted invasion of personal privacy]

4:40 p.m.–4:45 p.m.: *Reconciliation of ACRS Comments and Recommendations* (Open)—The Committee will discuss the responses from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports and letters.

5 p.m.–7 p.m.: *Preparation of ACRS Reports* (Open)—The Committee will discuss proposed ACRS reports.

Saturday, March 6, 2010, Conference Room T2-B1, Two White Flint North, Rockville, Maryland

8:30 a.m.–12:30 p.m.: *Preparation of ACRS Reports* (Open)—The Committee will continue its discussion of proposed ACRS reports.

12:30 p.m.–1 p.m.: *Miscellaneous* (Open)—The Committee will continue its discussion related to the conduct of Committee activities and specific issues that were not completed during previous meetings.

Procedures for the conduct of and participation in ACRS meetings were published in the **Federal Register** on October 14, 2009, (74 FR 52829-52830). In accordance with those procedures, oral or written views may be presented by members of the public, including representatives of the nuclear industry. Persons desiring to make oral statements should notify Mr. Derek Widmayer, Cognizant ACRS Staff, (Telephone: 301-415-7366, E-mail: Derek.Widmayer@nrc.gov), five days before the meeting, if possible, so that

appropriate arrangements can be made to allow necessary time during the meeting for such statements. In view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with the Cognizant ACRS staff if such rescheduling would result in major inconvenience.

Thirty-five hard copies of each presentation or handout should be provided 30 minutes before the meeting. In addition, one electronic copy of each presentation should be emailed to the Cognizant ACRS Staff one day before meeting. If an electronic copy cannot be provided within this timeframe, presenters should provide the Cognizant ACRS Staff with a CD containing each presentation at least 30 minutes before the meeting.

In accordance with Subsection 10(d) Public Law 92-463, and 5 U.S.C. 552b(c), certain portions of this meeting may be closed, as specifically noted above. Use of still, motion picture, and television cameras during the meeting may be limited to selected portions of the meeting as determined by the Chairman. Electronic recordings will be permitted only during the open portions of the meeting.

ACRS meeting agenda, meeting transcripts, and letter reports are available through the NRC Public Document Room at pdr.resource@nrc.gov, or by calling the PDR at 1-800-397-4209, or from the Publicly Available Records System component of NRC's document system which is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> or <http://www.nrc.gov/reading-rm/doc-collections/ACRS/>.

Video teleconferencing service is available for observing open sessions of ACRS meetings. Those wishing to use this service for observing ACRS meetings should contact Mr. Theron Brown, ACRS Audio Visual Technician (301-415-8066), between 7:30 a.m. and 3:45 p.m. (ET), at least 10 days before the meeting to ensure the availability of this service. Individuals or organizations requesting this service will be responsible for telephone line charges and for providing the equipment and facilities that they use to establish the video teleconferencing link. The availability of video teleconferencing services is not guaranteed.

Dated: February 17, 2010.

Andrew L. Bates,
Advisory Committee Management Officer.
[FR Doc. 2010-3489 Filed 2-22-10; 8:45 am]
BILLING CODE 7590-01-P

NUCLEAR REGULATORY COMMISSION

[NRC-2010-0002]

Sunshine Act; Notice of Meeting

DATES: Weeks of February 22, March 1, 8, 15, 22, 29, 2010.

PLACE: Commissioners' Conference Room, 11555 Rockville Pike, Rockville, Maryland.

STATUS: Public and Closed.

Week of February 22, 2010

Tuesday, February 23, 2010

9:25 a.m.

Affirmation Session (Public Meeting) (Tentative).

Entergy Nuclear Operations, Inc. (Indian Point Nuclear Generating Unit Nos. 1, 2, and 3); Docket Nos. 50-003-LT-2, 50-247-LT-2, 50-286-LT-2, and 72-51-LT-2. (Request for Hearing on Extension of Time to Complete License Transfer) (Tentative).

This meeting will be webcast live at the Web address—<http://www.nrc.gov>

9:30 a.m. Briefing on Decommissioning Funding (Public Meeting) (Contact: Thomas Fredrichs, 301-415-5971).

This meeting will be webcast live at the Web address—<http://www.nrc.gov>.

Week of March 1, 2010—Tentative

Tuesday, March 2, 2010

9:30 a.m. Briefing on Uranium Recovery (Public Meeting) (Contact: Dominick Orlando, 301-415-6749).

This meeting will be webcast live at the Web address—<http://www.nrc.gov>.

Week of March 8, 2010—Tentative

There are no meetings scheduled for the week of March 8, 2010.

Week of March 15, 2010—Tentative

Tuesday, March 16, 2010

1:30 p.m. Joint Meeting of the Federal Energy Regulatory Commission and the Nuclear Regulatory Commission on Grid Reliability (Public Meeting). (Contact: Kenn Miller, 301-415-3152).

This meeting will be webcast live at the Web address—<http://www.nrc.gov>.

Week of March 22, 2010—Tentative

There are no meetings scheduled for the week of March 22, 2010.

Week of March 29, 2010—Tentative

There are no meetings scheduled for the week of March 29, 2010.

* The schedule for Commission meetings is subject to change on short notice. To verify the status of meetings, call (recording)—(301) 415-1292. Contact person for more information: Rochelle Baval, (301) 415-1651.

Additional Information

The Briefing on Regional Programs—Programs, Performance, and Future Plans previously scheduled on Tuesday, February 9, 2010, at 9:30 a.m. has been postponed.

The NRC Commission Meeting Schedule can be found on the Internet at: <http://www.nrc.gov/about-nrc/policy-making/schedule.html>.

The NRC provides reasonable accommodation to individuals with disabilities where appropriate. If you need a reasonable accommodation to participate in these public meetings, or need this meeting notice or the transcript or other information from the public meetings in another format (e.g. braille, large print), please notify Angela Bolduc, Chief, Employee/Labor Relations and Work Life Branch, at 301-492-2230, TDD: 301-415-2100, or by e-mail at angela.bolduc@nrc.gov. Determinations on requests for reasonable accommodation will be made on a case-by-case basis.

This notice is distributed electronically to subscribers. If you no longer wish to receive it, or would like to be added to the distribution, please contact the Office of the Secretary, Washington, DC 20555 (301-415-1969), or send an e-mail to darlene.wright@nrc.gov.

February 18, 2010.

Rochelle C. Baval,
Office of the Secretary.

[FR Doc. 2010-3665 Filed 2-19-10; 4:15 pm]

BILLING CODE 7590-01-P

SECURITIES AND EXCHANGE COMMISSION

Sunshine Act Meeting

Notice is hereby given, pursuant to the provisions of the Government in the Sunshine Act, Public Law 94-409, that the Securities and Exchange Commission will hold a Closed Meeting on Thursday, February 25, 2010 at 2 p.m.

Commissioners, Counsel to the Commissioners, the Secretary to the



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, DC 20555 - 0001

February 16, 2010

AGENDA
570th ACRS MEETING
MARCH 4-6, 2010

THURSDAY, MARCH 4, 2010, CONFERENCE ROOM T-2B1, TWO WHITE FLINT NORTH,
ROCKVILLE, MARYLAND

- 1) ~~8:30~~ – 8:35 A.M. Opening Remarks by the ACRS Chairman (Open) (SAK/EMH)
1.1) Opening statement
1.2) Items of current interest
- 2) ~~8:35~~ – 10:00 A.M. Draft Final Interim Staff Guidance (ISG) on Fuel Cycle (ISG-7)
(Open) (CHB/CEA)
2.1) Remarks by the Subcommittee Chairman
2.2) Briefing by and discussions with representatives of the
NRC staff and the Nuclear Energy Institute (NEI) regarding
Draft Final ISG-7 on Fuel Cycle, the NRC staff's resolution
of public comments, and related matters

Representatives of the nuclear industry and members of the public
may provide their views, as appropriate.

~~10:00~~ – 10:15 A.M. *** BREAK ***

- 3) ~~10:15~~ – 12:00 P.M. Draft Final Regulatory Guide (RG) 1.141, "Containment Isolation
Provisions for Fluid Systems" (Open) (HBR/ZA)
3.1) Remarks by the Subcommittee Chairman
3.2) Briefing by and discussions with representatives of the
NRC staff regarding Draft Final RG 1.141, "Containment
Isolation Provisions for Fluid Systems," the NRC staff's
resolution of public comments, and related matters

Representatives of the nuclear industry and members of the public
may provide their views, as appropriate.

11:45
12:00 – 1:00 P.M. *** LUNCH ***

4) 1:00 – 2:00 P.M.

1:45

Draft Revision 1 to Regulatory Guide 4.11, "Terrestrial Environmental Studies for Nuclear Power Stations" (Open) (MTR/DAW)

- 4.1) Remarks by the Subcommittee Chairman
- 4.2) Briefing by and discussions with representatives of the NRC staff regarding Draft Revision 1 to RG 4.11, "Terrestrial Environmental Studies for Nuclear Power Stations," and related matters

Break 1:45-2:00

Representatives of the nuclear industry and members of the public may provide their views, as appropriate.

5) 2:00 – 3:15 P.M.

Status of Rulemaking for Disposal of Depleted Uranium and Other Unique Waste Streams (Open) (MTR/NMC)

- 5.1) Remarks by the Subcommittee Chairman
- 5.2) Briefing by and discussions with representatives of the NRC staff regarding the status of rulemaking efforts addressing disposal of depleted uranium and other unique waste streams, and related matters

Representatives of the nuclear industry and members of the public may provide their views, as appropriate.

✓ 3:15 – 3:30 P.M.

*** BREAK ***

6) 3:30 – 5:00 P.M.

Draft ACRS Report on the NRC Safety Research Program (Open) (DAP, et al/HPN, et al.)

- 6.1) Remarks by the Subcommittee Chairman
- 6.2) Discussion of a draft ACRS report on the NRC Safety Research Program

7) 5:00 – 7:00 P.M.

Preparation of ACRS Reports (Open)

Discussion of proposed ACRS reports on:

- 7.1) Draft Final Revision 1 to Regulatory Guide 1.62, "Manual Initiation of Protective Actions" (Open) (CHB/CEA/JCA)
- 7.2) Draft Final ISG-7 on Fuel Cycle (Open) (CHB/CEA)
- 7.3) Draft Final Regulatory Guide 1.141, "Containment Isolation Provisions for Fluid Systems" (Open) (HBR/ZA)
- 7.4) Draft Revision 1 to Regulatory Guide 4.11, "Terrestrial Environmental Studies for Nuclear Power Stations" (Open) (MTR/DAW)
- 7.5) Status of Rulemaking for Disposal of Depleted Uranium and Other Unique Waste Streams (Open) (MTR/NMC)

FRIDAY, MARCH 5, 2010, CONFERENCE ROOM T-2B1, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

8) ✓ 8:30 – 8:35 A.M.

Opening Remarks by the ACRS Chairman (Open) (SAK/CEA)

9) ✓ 8:35 – 10:15 A.M.

Digital Instrumentation and Control (I&C) Design Acceptance Criteria (DAC) Inspection Methodology (Open) (DCB/CEA)

- 9.1) Remarks by the Subcommittee Chairman
- 9.2) Briefing by and discussions with representatives of the NRC staff regarding Digital I&C DAC Inspection Methodology

Representatives of the nuclear industry and members of the public may provide their views, as appropriate.

off record → 10:00-10:30
harold ray 10: — 10:30
letter review
10) 10:15 – 10:30 A.M.

Break
*** BREAK ***

10) 10:30 – 12:00 A.M.

New Advanced Reactor Designs (Open) (DCB/KDW)

- 10.1) Remarks by the Subcommittee Chairman
- 10.2) Briefing by and discussions with representatives of the NRC staff regarding new advanced reactor designs such as NuScale, Iris, Babcock & Wilcox Modular, Hyperion, Toshiba's 4S, and General Electric's Prism

Representatives of the nuclear industry and members of the public may provide their views, as appropriate.

12:00 – 1:30 P.M.

*** LUNCH ***

11) 1:30 – 3:00 P.M.

Meeting with the NRC Executive Director for Operations (Open) (SAK/EMH)

- 11.1) Remarks by the ACRS Chairman
- 11.2) Briefing by and discussions with the NRC Executive Director for Operations (EDO) and Deputy EDOs on topics of mutual interest

12) 3:00 – 4:30 P.M.

Future ACRS Activities/Report of the Planning and Procedures Subcommittee (Open/Closed) (SAK/EMH)

- 12.1) Discussion of the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the Full Committee during future ACRS meetings
- 12.2) Report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, including anticipated workload and member assignments

[NOTE: A portion of this session may be closed pursuant to 5 U.S.C. 552b (c)(2) and (6) to discuss organizational and personnel matters that relate solely to internal personnel rules and practices of ACRS, and information the release of which would constitute a clearly unwarranted invasion of personal privacy.]

13) 4:30 – 4:45 P.M. Reconciliation of ACRS Comments and Recommendations (Open)
(SAK/CS/AFD)
Discussion of the responses from the NRC Executive Director for Operations to comments and recommendations included in recent ACRS reports and letters.

4:45 – 5:00 P.M. *** BREAK ***

14) 5:00 – 7:00 P.M. Preparation of ACRS Reports (Open)
7:40 pm Continue discussion of the proposed ACRS reports listed under Item 7.

SATURDAY, MARCH 6, 2010, CONFERENCE ROOM T-2B1, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

15) 8:30 – 12:30 P.M. Preparation of ACRS Reports (Open)
(10:00-10:15 A.M. BREAK) Continue discussion of the proposed ACRS reports listed under Item 7.

16) 12:30 – 1:00 P.M. Miscellaneous (Open) (SAK/EMH)
Discussion of matters related to the conduct of Committee activities and specific issues that were not completed during previous meetings, as time and availability of information permit.

NOTES:

- During the days of the meeting, phone number 301-415-7360 should be used in order to access anyone in the ACRS Office.
- Presentation time should not exceed 50 percent of the total time allocated for a given item. The remaining 50 percent of the time is reserved for discussion.
- Thirty five (35) hard copies and one (1) electronic copy of the presentation materials should be provided to the ACRS in advance of the briefing.
- One (1) electronic copy of each presentation should be emailed to the Designated Federal Official 1 day before the meeting. If an electronic copy cannot be provided within this timeframe, presenters should provide the Designated Federal Official with a CD containing each presentation at least 30 minutes before the meeting.

ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
570th FULL COMMITTEE MEETING

March 4-6, 2010

PLEASE PRINT

TODAY'S DATE: March 4, 2010

	<u>NAME</u>	<u>NRC ORGANIZATION</u>
1	CLIFF DOUTH	NRR/DLR
2	Dennis Daman	NMSS/FCSS
3	Patricia Silva	NMSS/FCSS
4	Marissa Bailey	NMSS/FCSS
5	WILKINS SMITH	NRR/DLR
6	Bob Dennis	NRR/DSS/SCVB
7	John N. Ridgely	RES/DE/RGDB
8	Brent Clayton	NRO/DSEB
9	Maice Lee	NMSS/FSME
10	RICHARD LEE	RES/DSA/PSTB
11	Brett Rini	RES/DE
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ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
570th FULL COMMITTEE MEETING

March 4-6, 2010

PLEASE PRINT

TODAY'S DATE: March 5, 2010

NAME

AFFILIATION

Craig Swanner
Shelby Small
Alan Lewis
Bob Hirmanpour
STEVEN MIRSKY
Kimberly Keithline
Scott Head
Michael Murray
JERRY WILSON
Andrea Sterdis
Mary Pietrzyk
JACK DONOHUE
Susan Vrahovitis
SHAWN MARSHALL

MPR Associates/TANE
AREVA
AREVA
NuStart
SAIC
NEI
STPNOC
STPNOC
NRC
TVA
NEI
NRC
NRC
NRC

ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
570th FULL COMMITTEE MEETING

March 4-6, 2010

PLEASE PRINT

TODAY'S DATE: March 4, 2010

	<u>NAME</u>	<u>AFFILIATION</u>
1	GORDON CLEFTON	NET
2	James Thompson	Hilti, Inc
3	Jim LIEBERMAN	TALISMAN
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LIST OF DOCUMENTS FROM THE
570th MEETING MARCH 3-4, 2010

Agenda Item 2:

Draft Final Interim Staff Guidance (ISG) on Fuel Cycle (ISG-7)

1. Table of Contents
2. Proposed Agenda
3. Background Material

Agenda Item 3:

Draft Final Regulatory Guide (RG) 1.141, "Containment Isolation Provisions for Fluid Systems"

4. Table of Contents
5. Purpose
6. Discussion
7. Conclusion
8. Reference
9. Regulatory Guide 1.141, "Containment Isolation Provision for Fluid Systems," June 2009 (After Public Comments)
10. Compare RG 1.141 June 2009 and RG 1.141 1
11. Applicable General Design Criteria for Containment Isolation
12. Subcommittee Status Report

Agenda Item 4:

Draft Revision 1 to Regulatory Guide 4.11, "Terrestrial Environmental Studies for Nuclear Power Stations"

13. Proposed Agenda
14. Status Report
15. Draft Regulatory Guide 4016, "*Terrestrial Environmental Studies for Nuclear Power Stations*"
16. Consultant Report – comments on DG-4106, "*Terrestrial Environmental Studies for Nuclear Power Stations.*"

Agenda Item 5:

Status of Rulemaking for Disposal of Depleted Uranium and Other Unique Waste Streams

17. Proposed Schedule
18. Status Report

LIST OF DOCUMENTS FROM THE
570th MEETING MARCH 3-4, 2010

Agenda Item 9:

Digital Instrumentation and Control (I&C) Design Acceptance Criteria (DAC) Inspection Methodology

- 19. Table of Contents
- 20. Proposed Agenda
- 21. Status Report

Agenda Item 10:

New Advanced Reactor Designs

- 22. Proposed Schedule
- 23. Status Report

Agenda Item 11:

Meeting with the NRC Executive Director for Operations

- 24. Proposed Schedule
- 25. Status Report
- 26. Background Material



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, DC 20555 - 0001**

March 16, 2010

**AGENDA
571st ACRS MEETING
APRIL 8-10, 2010**

**THURSDAY, APRIL 8, 2010, CONFERENCE ROOM T-2B1, TWO WHITE FLINT NORTH,
ROCKVILLE, MARYLAND**

- 1) 8:30 – 8:35 A.M. Opening Remarks by the ACRS Chairman (Open) (SAK/EMH)
 - 1.1) Opening statement
 - 1.2) Items of current interest

- 2) 8:35 – 10:00 A.M. Draft Final Interim Staff Guidance (ISG) DC/COL-ISG-016,
"Compliance with 10 CFR 50.54(hh)(2) and 10 CFR 52.80(d)"
(Open/Closed) (MVB/MB)
 - 2.1) Remarks by the Subcommittee Chairman
 - 2.2) Briefing by and discussions with representatives of the
NRC staff regarding Draft Final DC/COL-ISG-016,
"Compliance with 10 CFR 50.54(hh)(2) and 10 CFR
52.80(d)"

**[NOTE: A portion of this session may be closed to protect
unclassified safeguards information pursuant to 5 U.S.C. 552b
(c) (3).]**

10:00 – 10:15 A.M. * BREAK *****

- 3) 10:15 – 12:00 P.M. Selected Chapters of the Safety Evaluation Report (SER) with
Open Items Associated with the Review of the U.S. Evolutionary
Power Reactor (USEPR) Design Certification Application
(Open/Closed) (DAP/DAW)
 - 3.1) Remarks by the Subcommittee Chairman
 - 3.2) Briefing by and discussions with representatives of the
NRC staff and AREVA regarding Chapters 2, 4, 5, 8, 10,
12, and 17 of the SER with Open Items associated with the
review of the USEPR Design Certification Application

**[NOTE: A portion of this session may be closed to protect
information that is proprietary to AREVA NP and its
contractors pursuant to 5 U.S.C. 552b (c) (4).]**

12:00 – 1:00 P.M. * LUNCH *****

- 4) 1:00 – 4:00 P.M. Supplement 3 to General Electric (GE) Topical Report NEDC-33173PA, "Applicability of GE Methods to Expanded Operating Domains" (Open/Closed) (SB/ZA)
4.1) Remarks by the Subcommittee Chairman
4.2) Briefing by and discussions with representatives of the NRC staff and GE regarding Supplement 3 to GE Topical Report NEDC-33173PA, "Applicability of GE Methods to Expanded Operating Domains"

[NOTE: A portion of this session may be closed to protect information that is proprietary to GE and its contractors pursuant to 5 U.S.C. 552b (c) (4).]

4:00 – 4:15 P.M. * BREAK**

- 5) 4:15 – 7:00 P.M. Preparation of ACRS Reports
Discussion of proposed ACRS reports on:
5.1) Draft Final DC/COL-ISG-016, "Compliance with 10 CFR 50.54(hh)(2) and 10 CFR 52.80(d)" (Open/Closed) (MVB/MB)
5.2) Selected Chapters of the SER with Open Items Associated with the Review of the USEPR Design Certification Application (Open) (DAP/DAW)
5.3) Supplement 3 to GE Topical Report NEDC-33173PA, "Applicability of GE Methods to Expanded Operating Domains" (Open) (SB/ZA)

[NOTE: A portion of this session may be closed to protect unclassified safeguards information pursuant to 5 U.S.C. 552b (c) (3).]

FRIDAY, APRIL 9, 2010, CONFERENCE ROOM T-2B1, TWO WHITE FLINT NORTH, ROCKVILLE, MARYLAND

- 6) 8:30 – 8:35 A.M. Opening Remarks by the ACRS Chairman (Open) (SAK/EMH)
7) 8:35 – 9:30 A.M. Final ISG ESP/DC/COL-ISG-015, "Post-Combined License Commitments" (Open) (DCB/KH)
7.1) Remarks by the Subcommittee Chairman
7.2) Briefing by and discussions with representatives of the NRC staff regarding Final ESP/DC/COL-ISG-015, "Post-Combined License Commitments"

9:30 – 9:45 A.M. * BREAK *****

