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10 CFR 50.4
10 CFR 52.79

April 14, 2010

UN#10-097

ATTN: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

Subject: UniStar Nuclear Energy, NRC Docket No. 52-016
Response to Request for Additional Information for the
Calvert Cliffs Nuclear Power Plant, Unit 3,
RAI 227, Steam Generator Program

Reference: Surinder Arora (NRC) to Robert Poche (UniStar Nuclear Energy), "FINAL RAI
227 CIB1 4564" email dated April 8, 2010

The purpose of this letter is to respond to the request for additional information (RAI) identified in the NRC e-mail correspondence to UniStar Nuclear Energy, dated April 8, 2010 (Reference). This RAI addresses the Steam Generator Program, as discussed in Section 5.4 of the Final Safety Analysis Report (FSAR), as submitted in Part 2 of the Calvert Cliffs Nuclear Power Plant (CCNPP) Unit 3 Combined License Application (COLA), Revision 6.

The enclosure provides our response to RAI 227, Question 05.04.02.02-13, and includes revised COLA content. A Licensing Basis Document Change Request has been initiated to incorporate these changes into a future revision of the COLA.

Our response does not include any new regulatory commitments. This letter does not contain any sensitive or proprietary information.

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HRO

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If there are any questions regarding this transmittal, please contact me at (410) 470-4205, or Mr. Wayne A. Massie at (410) 470-5503.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on April 14, 2010

A handwritten signature in black ink, appearing to read 'Greg Gibson', with a long horizontal flourish extending to the right.

Greg Gibson

Enclosure: Response to NRC Request for Additional Information RAI 227, Question 05.04.02.02-13, Steam Generator Program, Calvert Cliffs Nuclear Power Plant, Unit 3

cc: Surinder Arora, NRC Project Manager, U.S. EPR Projects Branch
Laura Quinn, NRC Environmental Project Manager, U.S. EPR COL Application
Getachew Tesfaye, NRC Project Manager, U.S. EPR DC Application (w/o enclosure)
Loren Plisco, Deputy Regional Administrator, NRC Region II (w/o enclosure)
Silas Kennedy, U.S. NRC Resident Inspector, CCNPP, Units 1 and 2
U.S. NRC Region I Office

GTG/RDS/mdf

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Enclosure

**Response to NRC Request for Additional Information
RAI 227, Question 05.04.02.02-13, Steam Generator Program,
Calvert Cliffs Nuclear Power Plant, Unit 3**

RAI 227

Question 05.04.02.02-13

The response to RAI 40, Question 05.04.02.02-2, stated the CCNPP Unit 3 FSAR Table 13.4-1 would be modified to reference FSAR Section 5.4.2.5 for the Preservice Inspection Program. However, the reference to Section 5.4.2.5 was instead listed under Preservice Testing Program. Please revise FSAR Table 13.4-1 to move the Section 5.4.2.5 reference from "Preservice Testing Program" to "Preservice Inspection Program."

Response

The reference to FSAR 5.4.2.5, Steam Generator Program, that was inadvertently added to FSAR Table 13.4-1, Preservice Testing Program (Item 6) in the response to RAI 40, Question 05.04.02.02-2¹, will be moved to Table 13.4-1, Preservice Inspection Program (Item 4).

COLA Impact

FSAR Table 13.4-1 is being updated as follows:

¹ G. Gibson (UniStar Nuclear Energy) to Document Control Desk (U.S. NRC), Letter UN#09-004, Response to RAI No. 40, Revision 2, Standard Review Plan Section 05.04.02.02, Steam Generator Program, dated January 8, 2009.

Table 13.4-1—{Operational Programs Required by NRC Regulations and Program Implementation}

Item	Program Title	Source (Required By)	FSAR Section	Implementation Milestones	Requirements
1	In-service Inspection Program	10 CFR 50.55a(g)	3.8.1.7.2 5.2.4 6.6 5.4.2.5 Note 1	Prior to commercial service	10 CFR 50.55a(g) ASME XI IWA 2430
2	In-service Testing Program	10 CFR 50.55a(f); 10 CFR Part 50, App. A	3.9.6 5.2.4 Note 1	After generator online on nuclear heat	10 CFR 50.55a(f) ASME OM Code
3	Environmental Qualification Program	10 CFR 50.49(a)	3.11 Note 1	Prior to initial fuel load	License Condition
4	Preservice Inspection Program	10 CFR 50.55a(g)	3.8.1.7.2 5.2.4 6.6 <u>5.4.2.5</u> Note 1	Completion prior to initial plant startup	10 CFR 50.55a(g) ASME Code Section XI IWB-2200 IWC-2200, IWD-2200, IWE-2200, IWF-2210, and IWL-2210
5	Reactor Vessel Material Surveillance Program	10 CFR 50.60; 10 CFR 50, App. H	5.3.1 Note 1	Prior to initial fuel load	License Condition
6	Preservice Testing Program	10 CFR 50.55a(f)	3.9.6 5.2.4 5.4.2.5 Note 1	Prior to initial fuel load	License Condition
7	Containment Leakage Rate Testing Program	10 CFR 50.54(o); 10 CFR 50, App. A (GDC 53); 10 CFR 50, App. J	6.2.6 Note 1	Prior to initial fuel load	10 CFR50, App. J, Option B, Section III.A
8	Fire Protection Program	10 CFR 50.48	9.5.1 Note 1	Prior to initial fuel receipt for elements of the Fire Protection Program necessary to support receipt and storage of fuel onsite. Prior to initial fuel load for elements of the Fire Protection Program necessary to support fuel load and plant operation	License Condition