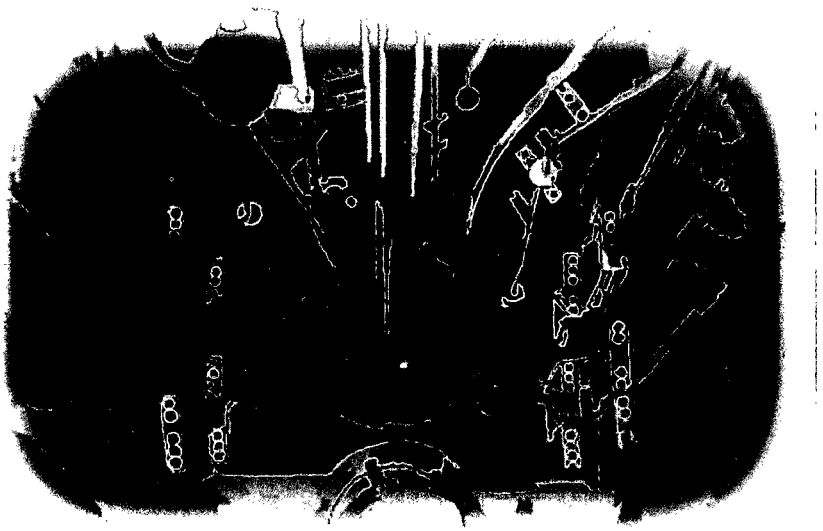
The University of Texas at Austin

Nuclear Engineering Teaching Laboratory



2009 Annual Report

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EXECUTIVE SUMMARY

The Nuclear Engineering Teaching Laboratory (NETL) facility supports the academic and research missions of The University of Texas, and has begun to provide these support functions to other institutions. The environmental research and analysis services performed by the NETL during have been used to support the Sandia National Laboratories, Los Alamos National Laboratory, Oak Ridge National Laboratory, the Canadian government, the National Oceanic and Atmospheric Administration, the University of Illinois, Texas A&M University and the State of Texas.

Department of Mechanical Engineering
THE UNIVERSITY OF TEXAS AT AUSTIN

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FORWARD

The mission of the Nuclear Engineering Teaching Laboratory at The University of Texas at Austin is to:

- **Educate the next generation of leaders in nuclear science and engineering.**
- **Conduct leading research at the forefront of the international nuclear community.**
- **Apply nuclear technology for solving multidisciplinary problems.**
- **Provide service to the citizens of Texas, the U.S., and the international community.**

This objective is achieved by carrying out a well-balanced program of education, research, and service. The NETL research reactor supports hands-on education in reactor physics and nuclear science. In addition, students in non-nuclear fields such as physics, chemistry, and biology use the reactor in laboratory course work. The NETL is also used in education programs for nuclear power plant personnel, secondary schools students and teachers, and the general public.

The NETL research reactor benefits a wide range of on-campus and off-campus users, including academic, medical, industrial, and government organizations. The principal services offered by our reactor involve material irradiation, trace element detection, material analysis, and radiographic analysis of objects and processes. Such services establish beneficial links to off-campus users, expose faculty and students to multidisciplinary research and commercial applications of nuclear science, and generate resources to help support Nuclear Engineering activities.

Steven Biegalski, Ph.D., P.E.
Director, Nuclear Engineering Teaching Laboratory

1.0 NUCLEAR ENGINEERING TEACHING LABORATORY ANNUAL REPORT

The Nuclear Engineering Laboratory Annual Report covers the period from January through December 2009. The report includes descriptions of the organization, NETL facilities, the reactor, experiment and research facilities and summaries of operations and radiological impact.

1.1 General

The NETL facility serves a multipurpose role, with the primary function as a "user facility" for faculty, staff, and students of the Cockrell School of Engineering. The NETL supports development and application of nuclear methods for researchers from other universities, government organizations and industry. The NETL provides nuclear analytic services to researchers, industry, and other laboratories for characterization, testing and evaluation of materials. The NETL provides public education through tours and demonstrations.

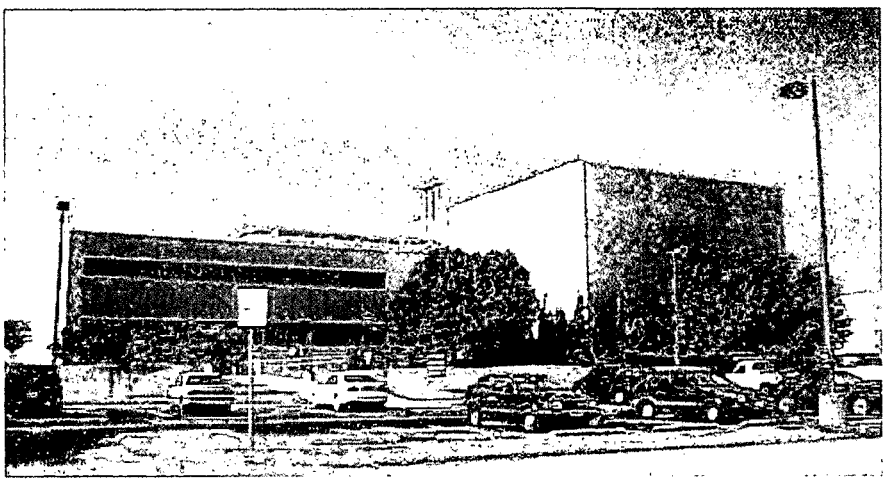

Figure 1-1 NETL - Nuclear Engineering Teaching Laboratory

Activities at NETL are regulated by Federal and State agencies. The nuclear reactor is subject to the terms and specifications of Nuclear Regulatory Commission (NRC) License R-129, a class 104 research reactor license. A second NRC license for special nuclear materials, SNM-180, authorizes possession of a subcritical assembly, neutron sources, and various equipment. The

NETL is responsible for administration and management of both licenses. Activities at the University using radioisotopes are conducted under a license by the State of Texas, L00485. Functions of the broad license are the responsibility of the University Office of Environmental Health and Safety.

1.2 Purpose of this Report

This report meets requirements of the reactor Technical Specifications and the Department of Energy Fuels Assistance program, and provides an overview of the education, research, and service programs of the NETL for the calendar year 2009.

1.2.1 TRIGA II Reactor Technical Specifications

The NETL TRIGA II reactor Technical Specifications (section 6.6.1) requires submission of an annual report to the Nuclear Regulatory Commission. Table 1.1 correlates specified requirements to the report.

Table 1.1, TRIGA Mark II Technical Specification and the Annual Report

Specification	Section
A narrative summary of reactor operating experience including the energy produced by the reactor or the hours the reactor was critical, or both.	5.0, 6.1, 6.3
The unscheduled shutdowns & corrective action taken to preclude recurrence	6.2
Major preventive & corrective maintenance operations with safety significance	6.4
Major changes in the reactor facility and procedures, tabulation of new tests or experiments, or both, significantly different from those performed previously, including conclusions that no unreviewed safety questions were involved	6.6
A summary of radioactive effluents (nature & amount) released or discharged to the environs beyond effective control of the university as determined at or before the point of such release or discharge, including to the extent practicable an estimate of individual radionuclides present in the effluent or a statement that the estimated average release after dilution or diffusion is less than 25% of the concentration allowed or recommended	7.2
A summary of exposures received by facility personnel and visitors where such exposures are greater than 25% of that allowed or recommended.	7.3
A summarized result of environmental surveys performed outside the facility	7.4

1.2.2 The Department of Energy Fuels Assistance Program

The DOE University Fuels Assistance program (DE-AC07-05ID14517, subcontract 00078206, 08/01/2008-08/31/2013) supports the facility for utilization of the reactor in a program of education and training of students in nuclear science and engineering, and for faculty and student research. The contract requires an annual progress report in conjunction with submittal of a Material Balance Report and Physical Inventory Listing report. Specific technical details of the report (listed in Table 2.2) are sent under separate cover to the DOE with this Annual Report.

Table 2.2, DOE Reactor Fuel Assistance Report Requirements

Fuel usage (grams Uranium 235 & number of fuel elements)
Inventory of unirradiated fuel elements in storage
Inventory of fuel elements in core
Inventory of useable irradiated fuel elements outside of core
Projected 5-year fuel needs
Current inventory of other nuclear material items with DOE-ID project identifier (i.e., "J")
Point of contact for nuclear material accountability

2.0 ORGANIZATION AND ADMINSTRATION

The University of Texas System (UTS) was established by the Texas Constitution in 1876, and currently consists of nine academic universities and six health institutions. The UTS mission is to provide high-quality educational opportunities for the enhancement of the human resources of Texas, the nation, and the world through intellectual and personal growth.

The Board of Regents is the governing body for the UTS. It is composed of nine members who are appointed by the Governor and confirmed by the Senate. Terms are of six years each and staggered, with the terms of three members expiring on February 1 of odd-numbered years. Current members of the current Board of Regents are listed in Table 2.1.

Table 2.1

The University of Texas Board of Regents for 2009
James R. Huffines, Chairman
Paul Foster, Vice Chairman
Colleen McHugh, Vice Chairman
Francie A. Frederick, General Counsel to the Board of Regents
R. Steven Hicks, Austin

http://registrar.utexas.edu/catalogs/gi09-10/ch00/gi09.regents.html, revised August 2009

The chief executive officer of the UTS is the Chancellor. The Chancellor has direct line responsibility for all aspects of UTS operations, and reports to and is responsible to the Board of Regents. The current Chancellor and Staff are listed in Table 2.2.

Table 2.2

University of Texas System Personnel
Francisco G. Cigarroa, MD, Chancellor
David B. Prior, PhD, Executive Vice Chancellor for Academic Affairs
Scott C. Kelley, PhD, Executive Vice Chancellor for Business Affairs
Kenneth I. Shine, MD, Executive Vice Chancellor for Health Affairs
Tonya Moten Brown, JD, Vice Chancellor for Administration
Barry D. Burgdorf, JD, Vice Chancellor and General Counsel
Randa S. Safady, PhD, Vice Chancellor for External Relations
William H. Shute, JD, Vice Chancellor for Federal Relations
Barry McBee, JD, Vice Chancellor for Governmental Relations
Keith McDowell, PhD, Vice Chancellor for Research and Technology Transfer
Francie A. Frederick, JD, General Counsel to the Board of Regents

http://registrar.utexas.edu/catalogs/gi09-10/ch00/gi09.officers.html, revised August 2009

UT Austin is the flagship campus of the UTS. The facility operating license for the TRIGA Mark II at the NETL is issued to the University of Texas at Austin. Figure 2-1 reflects the organizational structure for 4 levels of line management of the NETL reactor, as identified in the Technical Specifications, as well as oversight functions. Other NETL resources (in addition to line management positions) include staff with specialized functions, and faculty and facility users. NETL support is through a combination of State allocation, research programs, and remuneration for service.

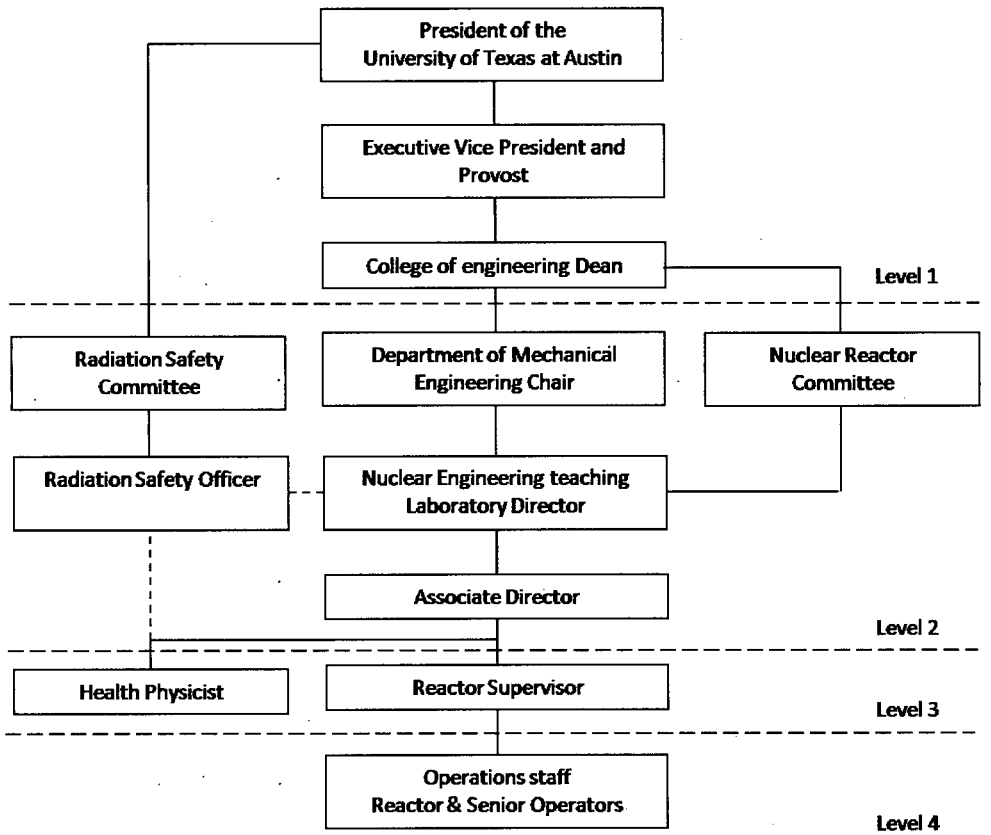

Figure 2-1, Organizational Structure for the University of Texas at Ausitn TRIGA Reactor

2.1 Level 1 Personnel

Level 1 represents the central administrative functions of the University and the Cockrell School of Engineering. The University of Texas at Austin is composed of 16 separate colleges and schools; the Cockrell School of Engineering manages eight departments with individual degree programs. The Nuclear Engineering Teaching Laboratory (NETL) is one of several education and research functions within the School. Current Level 1 personnel are reported in Table 2.3.

2.1.1 President, University of Texas at Austin

The President is the individual vested by the University of Texas system with responsibility for the University of Texas at Austin.

2.1.2 Executive Vice president and Provost (Provost)

Research and educational programs are administered through the Office of the Executive Vice President and Provost. Separate officers assist with the administration of research activities and academic affairs with specific management functions delegated to the Dean of the Cockrell School of Engineering and the Chairman of the Mechanical Engineering Department.

2.1.3 Dean of the Cockrell School of Engineering

The Dean of the Cockrell School of Engineering reports to the Provost. The School consists of 8 departments and undergraduate degree programs and 12 graduate degree programs.

2.1.4 Department of Mechanical Engineering Chairman

The Chairman reports to the Dean of the Cockrell School of Engineering. The Department manages 8 areas of study, including Nuclear and Radiation Engineering.

Table 2.3

The University of Texas at Austin Administration (Level 1)
William Powers Jr., JD, President
Steven W. Leslie, PhD, Executive Vice President and Provost
Gregory L. Fenves, PhD, Dean, Cockrell School of Engineering
Joseph J. Beaman, Chairman of Department of Mechanical Engineering

http://registrar.utexas.edu/catalogs/gi09-10/ch00/gi09.officers.html, revised August 2009

2.2 Level 2 Personnel

The Nuclear Engineering Teaching Laboratory operates as a unit of the Department of Mechanical Engineering at The University of Texas. Level 2 personnel are those with direct responsibilities for administration and management of resources for the facility, including the Chair of the Mechanical Engineering Department, the NETL Director and Associate Director. Oversight roles are provided at Level 2 by the Radiation Safety Committee, the Radiation Safety Officer and the Nuclear Reactor Committee. The current complement of Level 2 personnel is reported along with the NETL facility staff and the Nuclear and Radiation Engineering program faculty in Table 2.4.

Table 2.4

Facility Staff & NRE Faculty		
NETL Facility Staff		NRE Faculty
Director	S. Biegalski	S. Biegalski
Associate Director	D. S. O'Kelly to 6/09 P. M. Whaley of 1/10	S. Landsberger E. Schneider
Reactor Supervisor	M. Krause	M. Deinhert
Health Physicist & Lab manager	D. J. O'Kelly to 6/09 T. Tipping as of 10/09	
Administrative Associate	D. Judson	
Electronics Technician/ Reactor Operator	L. Welch	
Reactor Operators	H. Fuentes L. Patterson S. Robinson	
Health Physics Technician	J. Sims	

2.2.1 Director, Nuclear Engineering Teaching Laboratory (NETL Director)

Nuclear Engineering Teaching Laboratory programs are directed by an engineering faculty member with academic responsibilities in nuclear engineering and research related to nuclear

applications. The Director is a member of the Cockrell School of Engineering, and the Department of Mechanical Engineering.

2.2.2 Associate Director

The Associate Director is responsible for safe and effective conduct of operations and maintenance of the TRIGA nuclear reactor. Other activities performed by the Associate Director and staff include neutron and gamma irradiation service, operator/engineering training courses, and teaching reactor short courses. In addition to Level 3 staff, an Administrative Assistant and an Electronics Technician report to the Associate Director. Many staff functions overlap, with significant cooperation required.

2.2.4 Safety Oversight

Safety oversight is provided for radiation protection and facility safety functions. A University of Texas Radiation Safety Committee is responsible programmatically for coordination, training and oversight of the University radiation protection program, with management of the program through a Radiation Safety Officer. Current personnel on the Radiation Safety Committee are listed on Table 2.5.

Nuclear reactor facility safety oversight is the responsibility of a Nuclear Reactor Committee; a request has been made to the Nuclear Regulatory Commission to change the name "Nuclear Reactor Committee" to "Reactor Oversight Committee" to better describe the committee function for the University and avoid confusion with other NRC organizations. "Reactor Oversight Committee" will be used in this report pending approval. Current personnel on the Reactor Oversight Committee are listed on Table 2.6.

Radiation Safety Committee. The Radiation Safety Committee reports to the President and has the broad responsibility for policies and practices regarding the license, purchase, shipment, use, monitoring, disposal and transfer of radioisotopes or sources of ionizing radiation at The University of Texas at Austin. The Committee meets at least three times each calendar year. The

Committee is consulted by the Office of Environmental Health and Safety concerning any unusual or exceptional action that affects the administration of the Radiation Safety Program.

Table 2.5

Radiation Safety Committee 2009-2010
Gerald W. Hoffmann, Ph.D., *Chair,* Department of Physics
Juan M. Sanchez, Ph.D., *Vice Chair,* Vice President for Research
Neal Armstrong, Ph.D., Vice Provost
Steven R. Biegalski, Ph.D., Director, Nuclear Engineering Teaching Laboratory
Gerald R. Harkins, *ex-officio,* Associate Vice President, Campus Safety and Security
W. Scott Pennington, *ex-officio,* Office of Environmental Health & Safety
Jon D. Robertus, Ph.D., Department of Chemistry & Biochemistry
Bob G. Sanders, Ph.D., School of Biological Sciences

http://www.utexas.edu/research/resources/committees.php#rsc, Revised Jan 2010

Radiation Safety Officer. A Radiation Safety Officer holds delegated authority of the Radiation Safety Committee in the daily implementation of policies and practices regarding the safe use of radioisotopes and sources of radiation as determined by the Radiation Safety Committee. Radiation Safety Officer responsibilities are outlined in *The University of Texas at Austin Radiation Safety Manual.* The Radiation Safety Officer has an ancillary function reporting to the NETL Director as required on matters of radiological protection. The Radiation Safety Program is administered through the University Office of Environmental Health and Safety.

A NETL Health Physicist (Level 3) manages daily radiological protection functions at the NETL, and reports to the Radiation Safety Officer as well as the Associate Director. This arrangement assures independence of the Health Physicist through the Radiation Safety Officer while maintaining close interaction with NETL line management.

Reactor Oversight Committee (ROC). The Reactor Oversight Committee (formerly known as the Nuclear Reactor Committee) evaluates, reviews, and approves facility standards for safe operation of the nuclear reactor and associated facilities. The ROC meets at least semiannually. The ROC provides reports to the Dean on matters as necessary throughout the year and submits a final report of activities no later than the end of the spring semester. The ROC makes recommendations to the NETL Director for enhancing the safety of nuclear reactor operations.

Specific requirements in the Technical Specifications are incorporated in the committee charter, including an audit of present and planned operations. The ROC is chaired by a professor in the Cockrell School of Engineering. ROC Membership varies, consisting of ex-officio and appointed positions. The Dean appoints at least three members to the Committee that represent a broad spectrum of expertise appropriate to reactor technology, including personnel external to the School.

Table 2.6

Reactor Oversight Committee 2009-2010
Howard Liljestrand (CAEE), Chair
Lynn Katz (CAEE)
Steven Biegalski (ME)
Erich Schneider (ME)
Joseph Beaman, ex-officio (ME)
Michael Krause, ex-officio (NETL)
D. J. O'Kelly, ex-officio (NETL) through Jun 2009
Tracy Tipping, ex-officio (NETL) as of Oct 2009
D. S. O'Kelly, ex-officio (NETL) through Jun 2009
Mike Whaley, ex-officio (NETL) as of Jan 2010
John G. Ekerdt, ex-officio
Scott Pennington, other (Radiation Safety Officer)
Lawrence R. Jacobi (External Representative)
Jodi Jenkins (External Representative)

http://www.engr.utexas.edu/faculty/committees/225-roc

2.3 Level 3 Personnel

Level 3 personnel are responsible for managing daily activities at the NETL. The Reactor Supervisor and Health Physicist are Level 3. The current Reactor Supervisor and Health Physicist are listed on Table 2.4.

2.3.1 Reactor Supervisor

The Reactor Supervisor function is incorporated in a Reactor Manager position, responsible for daily operations, maintenance, scheduling, and training. The Reactor Manager is responsible for the maintenance and daily operations of the reactor, including coordination and performance of activities to meet the Technical Specifications of the reactor license. The Reactor Manager plans

and coordinates emergency exercises with first responders and other local support (Austin Fire Department, Austin/Travis County EMS, area hospitals, etc.).

The Reactor Manager, assisted by Level 4 personnel and other NETL staff, implements modifications to reactor systems and furnishes design assistance for new experiment systems. The Reactor Manager assists initial experiment design, fabrication, and setup. The Reactor Manager provides maintenance, repair support, and inventory control of computer, electronic, and mechanical equipment. The Administrative Assistant and Reactor Manager schedule and coordinate facility tours, and support coordination of building maintenance.

2.2.1 Health Physicist

The Health Physicist function is incorporated into a Laboratory Manager position, responsible for radiological protection (Health Physics), safe and effective utilization of the facility (Lab Management), and research support. Each of these three functions is described below. The Laboratory Manager is functionally responsible to the NETL Associate Director, but maintains a strong reporting relationship to the University Radiation Safety Officer and is a member of the Radiation Safety Committee. This arrangement allows the Health Physicist to operate independent of NETL operational constraints in consideration of radiation safety.

Health Physics. NETL is a radiological facility operating in the State of Texas under a facility operating license issued by the Nuclear Regulatory Commission (NRC). Radioactive material and activities associated with operation of the reactor are regulated by the NRC, and the uses of radioactive materials at the NETL not associated with the reactor are regulated by the Texas Department of State Health Services (TDSHS) Radiation Control Program. The NETL Health Physicist ensures operations comply with these requirements, and that personnel exposures are maintained ALARA ("as low as is reasonably achievable"). One or more part-time Undergraduate Research Assistant (URA) may assist as Health Physics Technicians.

Lab Management. The lab management function is responsible for implementation of occupational safety and health programs at the NETL. The Laboratory Manager supports University educational activities through assistance to student experimenters in their projects by

demonstration of the proper radiation work techniques and controls. The Laboratory Manager participates in emergency planning for NETL and the City of Austin to provide basic response requirements and conducts off-site radiation safety training to emergency response personnel such as the Hazardous Materials Division of the Fire Department, and Emergency Medical Services crews.

Research Support. The mission of The University of Texas at Austin is to achieve excellence in the interrelated areas of undergraduate education, graduate education, research and public service. The Laboratory Manager and research staff supports the research and educational missions of the university at large, as well as development or support of other initiatives.

The Laboratory Manager is responsible for coordinating all phases of a project, including proposal and design, fabrication and testing, operation, evaluation, and removal/dismantlement. Researchers are generally focused on accomplishing very specific goals, and the research support function ensures the NETL facilities are utilized in a safe efficient manner to produce quality data. The Laboratory Manager obtains new, funded research programs to promote the capabilities of the neutron beam projects division for academic, government and industrial organizations and/or groups.

The NETL provides unique facilities for nuclear analytic techniques, including but not limited to elemental analysis (instrumental neutron activation analysis, prompt gamma analysis), measurements of physical characteristics (neutron depth profiling, neutron radiography) and experimental techniques investigating fundamental issues related to nuclear physics and condensed matter. Nuclear analytical techniques support individual projects ranging from class assignments to measurements for faculty research.

The Laboratory Manager manages the use of the five beam ports with the Texas Cold Neutron Source, Neutron Depth Profiling, Neutron Guide and Focusing System, Prompt Gamma Activation Analysis Neutron Radiography and Texas Intense Positron Source. Projects are supported in engineering, chemistry, physics, geology, biology, zoology, and other areas. Research project support includes elemental measurements for routine environmental and

innovative research projects. The neutron activation analysis technique is made available to different state agencies to assist with quality control of sample measurements.

2.4 Level 4 Personnel

Reactor Operators and Senior Reactor Operators (RO/SRO) operate and maintain the reactor and associated facilities. An RO/SRO may operate standard reactor experiment facilities as directed by the Reactor Supervisor.

2.5 Other Facility Staff

In addition to the line management positions defined in Figure 2-1, NETL staff includes an Administrative Assistant, and Electronics Technician, and variously one or more Undergraduate Research Assistants assigned either non-licensed maintenance support (generally but not necessarily in training for Reactor Operator licensure) or to support the Laboratory Manager as Health Physics Technicians and/or research support.

2.6 Faculty and Facility Users

The complement of faculty and facility users at the NETL is extremely variable. Functionally faculty and facility users are associated with the NETL in the capacity of academic utilization, other educational efforts, or research & service. A description of these activities follows.

2.6.1 Academic Utilization

The NETL is integrated in the Nuclear and Radiation Engineering program (NRE) of Mechanical Engineering (ME). The ME faculty complement directly supporting the nuclear education program is listed in Table 2.7. Successful participation in the undergraduate program results in a Bachelor of Science in Mechanical Engineering, Nuclear Engineering certification; the degree is essentially a major in Mechanical Engineering with a minor in Nuclear Engineering. All Mechanical Engineering degree requirements must be met with an additional set of specific nuclear engineering courses successfully completed.

Table 2.7

University of Texas Nuclear and Radiation Engineering program Faculty
Dr. Sheldon Landsberger, Nuclear and Radiation Engineering Professor
Dr. Steven Biegalski, Nuclear and Radiation Engineering Associate Professor
Dr. Erich Schneider, Nuclear and Radiation Engineering Assistant Professor
Dr. Ofodike A. Ezekoye, Thermal Fluids Systems Professor
Dr. John R. Howell, Thermal Fluids Systems Professor
Dr. Dale Klein, Nuclear and Radiation Engineering (on leave)
Dr. Kendra M. Foltz-Biegalski, Nuclear and Radiation Engineering Research Engineer
Dr. Elmira Popova, Operations Research Associate Professor
Dr. David Hearnsberger, Lecturer
Dr.Mark Deinert, Nuclear and Radiation Engineering, Thermal Fluid Systems, Assistant Professor
Dr.Mitch Pryor, Robotics Research Group Research Associate

Of the five undergraduate Nuclear Engineering courses and the dozen graduate Nuclear Engineering courses, five courses make extensive use of the reactor facility. Table 2.8 lists the courses currently in the UT course catalog that use the reactor and its experiment facilities.

Table 2.3, Nuclear Engineering Courses

Undergraduate
ME 136N, 236N: Concepts in Nuclear and Radiological Engineering
ME 337C: Introduction to Nuclear Power Systems
ME337F: Nuclear Environmental Protection
ME 337G: Nuclear Safety and Security
ME 361E: Nuclear Operations and Reactor Engineering
ME 361F: Radiation and Radiation protection Laboratory
Graduate
ME 388C: Nuclear Power Engineering
ME 388D: Nuclear Reactor Theory I
ME 388F: Computational Methods in Radiation Transport
ME 388G: Nuclear Radiation Shielding
ME 388H: Nuclear Safety and Security
ME 388J: Neutron Interactions and their Applications in Nuclear Science and Engineering
ME 388M: Mathematical Methods for Nuclear and Radiation Engineers
ME 388N: Design of Nuclear Systems I
ME 388P: Applied Nuclear Physics
ME 388S: Modern Trends in Nuclear and Radiation Engineering
ME 389C: Nuclear Environmental Protection
NE 389F: The Nuclear Fuel Cycle
ME 390F: Nuclear Analysis Techniques
ME 390G: Nuclear Engineering Laboratory
ME390T: Nuclear- and Radio-Chemistry

The NRE program's graduate degrees are completely autonomous; they are Master of Science in Engineering (Concentration in Nuclear Engineering) and Doctor of Philosophy (Concentration in Nuclear Engineering). Course requirements for these degrees and the qualifying examination for the Ph.D. are separate and distinct from other areas of Mechanical Engineering. A Dissertation Proposal and Defense of Dissertation are required for the Ph.D. degree and acted on by a NRE dissertation committee.

The Nuclear and Radiological Engineering program faculty in 2009 initiated a Summer Nuclear Engineering Institute (SNEI), with credit hours extended to participants. Initial program enrollment was fifteen students from schools around the country. This one month institute is designed for sophomores and juniors majoring in technical fields outside of nuclear engineering interested in a career or graduate school in the field. SNEI students study fundamental nuclear engineering concepts, and gain hands-on experience at the research reactor and other experiment facilities.

2.6.2 Other Education Efforts

The NETL hosted an education program for members of the Nuclear Regulatory Commission in 2009. The NETL has participated in the IAEA Fellowship programs for the past decade. Several Fellows and Visiting Scientists spend 3-6 months at the NETL per year.

In addition to formal classes, the NETL routinely provides short courses or tours for Texas agencies, high schools and pre-college groups such as the Boy Scouts of America. Tours and special projects are available to promote public awareness of nuclear energy issues. A typical tour is a general presentation for high school and civic organizations. Other tours given special consideration are demonstrations for interest groups such as physics, chemistry and science groups.

2.6.3 Research & Service

A more comprehensive description of the nuclear analytic techniques and facilities available at the NETL is provided in section 5. Personnel support for these activities includes faculty, graduate and undergraduate research assistants, and NETL staff.

2.7 NETL Support

NETL funding is provided by state appropriations, research grants, and fees accrued from service activities. Research funding supplements the base budget provided by the State and is generally obtained through competitive research and program awards. Funds from service activities supplement base funding to allow the facility to provide quality data acquisition and analysis capabilities. Both sources of supplemental funds (competitive awards and service work) are important to the education and research environment for students. The U.S. Nuclear Regulatory Commission supported development of the Summer Nuclear Engineering Institute, and supports continuation of the program.

3.0 FACILITY DESCRIPTION

3.1 NETL History

Development of the nuclear engineering program was an effort of both physics and engineering faculty during the late 1950's and early 1960's. The program became part of the Mechanical Engineering Department where it currently resides. The program installed and operated the first UT TRIGA nuclear reactor in Taylor Hall on the main campus. Initial criticality for the first UT reactor was August 1963. Power at startup was 10 kilowatts with a power upgrade to 250 kilowatts in 1968. Total burnup during the 25 year period from 1963 to final operation in April 1988 was 26.1 megawatt-days. Pulse capability of the reactor was 1.4% $\Delta k/k$ with a total of 476 pulses during the operating history.

In October 1983, planning was initiated for the NETL to replace the original UT TRIGA installation. Construction was initiated December 1986 and completed in May 1989. The NETL facility operating license was issued in January 1992, with initial criticality on March 12, 1992. Dismantling and decommissioning of the first UT TRIGA reactor facility was completed in December 1992.

3.2 NETL SITE, J.J. Pickle Research Campus

Land development in the area of the current NETL installation began as an industrial site during the 1940's. Following the 1950's, lease agreements between the University and the Federal government led to the creation of the Balcones Research Center. The University became owner of the site in the 190's, and in 1994 the site name was changed to the J.J. Pickle Research Campus (PRC) in honor of retired U.S. Congressman James "Jake" Pickle.

The PRC is a multidiscipline research campus on 1.87 square kilometers. The site consists of two approximately equal areas, east and west. An area of about 9000 square meters on the east tract is the location of the NETL building. Sixteen separate research units and at least five other academic research programs conduct research at locations on the PRC. Adjacent to the NETL site are the Center for Research in Water Resources, the Bureau of Economic Geology, and the

Research Office Complex, illustrating the diverse research activities on the campus. A Commons Building provides cafeteria service, recreation areas, meeting rooms, and conference facilities.

3.3 NETL Building Description

The NETL building is a 1950 sq meter (21,000 sq ft), facility with laboratory and office spaces. Building areas consist of two primary laboratories of 330 sq m (3600 sq ft) and 80 sq m (900 sq ft), eight support laboratories (217 sq m, 2340 sq ft), and six supplemental areas (130 sq m, 1430 sq ft). Conference and office space is allocated to 12 rooms totaling 244 sq m (2570 sq ft). One of the primary laboratories contains the TRIGA reactor pool, biological shield structure, and neutron beam experiment area. A second primary laboratory consists of 1.3 meter (4.25 ft) thick walls for use as a general purpose radiation experiment facility. Other areas of the building include shops, instrument & measurement laboratories, and material handling facilities.

The NETL Annex was installed in 2005, a 24 by 60 foot modular class room building adjacent to the NETL building. The building provides classroom space and offices for graduate students working at the NETL.

4.0 UT-TRIGA MARK II RESEARCH REACTOR

TRIGA is an acronym for Training, Research, Isotope production, General Atomics. The TRIGA Mark II reactor is a versatile and inherently safe research reactor conceived and developed by General Atomics to meet education and research requirements. The UT-TRIGA reactor provides sufficient power and neutron flux for comprehensive and productive work in many fields including physics, chemistry, engineering, medicine, and metallurgy

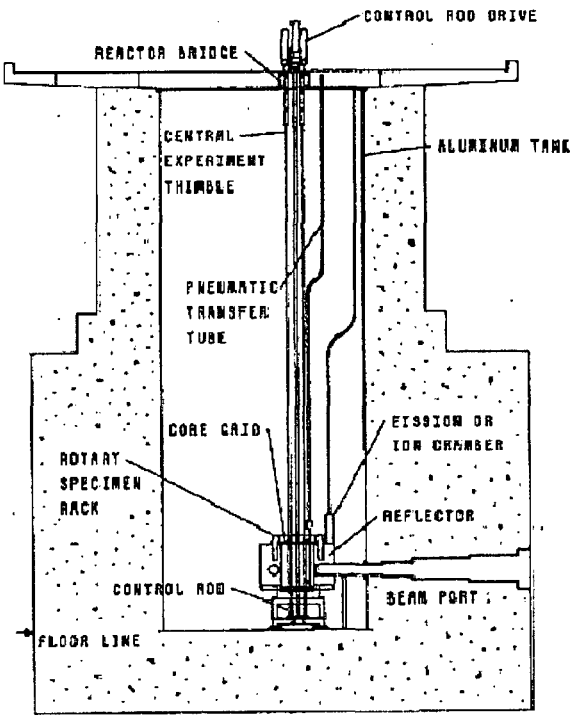

Figure 4-1, UT TRIGA Mark II Nuclear Research Reactor

The NETL UT-TRIGA reactor is an above-ground, fixed-core research reactor. The reactor core is located at the bottom of an 8.2 meter deep water-filled tank surrounded by a concrete shield structure. The water serves as a coolant, neutron moderator, and transparent radiation shield. The reactor core is surrounded by a reflector, a 1 foot thick graphite cylinder. The reactor is controlled by manipulating cylindrical "control rods" containing boron.

4.1 Reactor Core.

The reactor core is an assembly of about 100 fuel elements surrounded by an annular graphite neutron reflector. Fuel elements are positioned by an upper and lower grid plate, with penetrations of various sizes in the upper grid plate to allow insertion of experiments. Each fuel element consists of a fueled region with graphite sections at top and bottom, contained in a thin-walled stainless steel tube. The fuel region is a metallic alloy of low-enriched uranium in a zirconium hydride (UZrH) matrix. Physical properties of the TRIGA fuel provide an inherently safe operation. Rapid power transients to high powers are automatically suppressed without using mechanical control; the reactor quickly and automatically returns to normal power levels. Pulse operation, a normal mode, is a practical demonstration of this inherent safety feature.

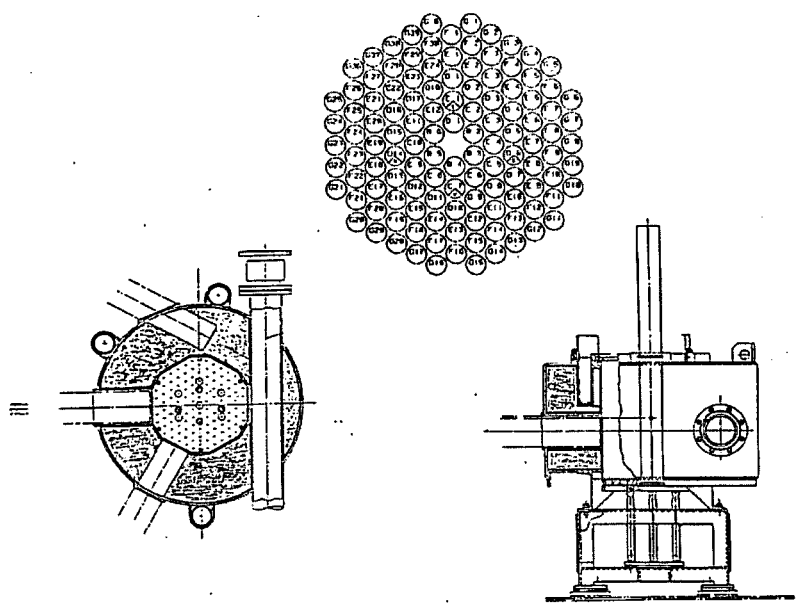

Figure 4-2, Core and Support Structure Details

4.2 Reactor Reflector.

The reflector is a graphite cylinder in a welded aluminum-canister. A 10" well in the upper surface of the reflector accommodates an irradiation facility, the rotary specimen rack (RSR), and horizontal penetrations through the side of the reflector allow extraction of neutron beams. In

2000 the canister was flooded to limit deformation stemming from material failure in welding joints. In 2004, the reflector was replaced with some modification, including a modification to the upper grid plate for more flexible experiment facilities.

4.3 Reactor Control.

The UT-TRIGA research reactor can operate continuously at nominal powers up to 1 MW, or in the pulsing mode with maximum power levels up to 1500 MW for durations of about 10 msec. The pulsing mode is particularly useful in the study of reactor kinetics and control. The UT-TRIGA research reactor uses a compact microprocessor-driven control system. The digital control system provides a unique facility for performing reactor physics experiments as well as reactor operator training. This advanced system provides for flexible and efficient operation with precise power level and flux control, and permanent retention of operating data.

The power level of the UT-TRIGA is controlled by a regulating rod, two shim rods, and a transient rod. The control rods are fabricated with integral extensions containing fuel (regulating and shim rods) or air (transient rod) that extend through the lower grid plate for full span of rod motion. The regulating and shim rods are fabricated from B_4C contained in stainless steel tubes; the transient rod is a solid cylinder of borated graphite clad in aluminum. Removal of the rods from the core allows the rate of neutron induced fission (power) in the UZrH fuel to increase. The regulating rod can be operated by an automatic control rod that adjusts the rod position to maintain an operator-selected reactor power level. The shim rods provide a coarse control of reactor power. The transient rod can be operated by pneumatic pressure to permit rapid changes in control rod position. The transient rod moves within a perforated aluminum guide tube.

5.0 EXPERIMENT AND RESEARCH FACILITIES

Neutrons produced in the reactor core can be used in a wide variety of research applications including nuclear reaction studies, neutron scattering experiments, nuclear analytical techniques, and irradiation of samples. Facilities for positioning samples or apparatus in the core region include cut-outs fabricated in the upper grid plate, a central thimble in the peak flux region of the core, a rotary specimen rack in the reactor graphite reflector, and a pneumatically operated transfer system accessing the core in an in-core section. Beam ports, horizontal cylindrical voids in the concrete shield structure, allow neutrons to stream out away from the core. Experiments may be performed inside the beam ports or outside the concrete shield in the neutron beams. Areas outside the core and reflector are available for large equipment or experiment facilities. Projects currently being performed by NRE and NETL personnel are listed in Table 5.1, and Table 5.2 provides a tabulation of current publications and presentations (both at the end of this section).

Figure 5-1 provides the number of hours of reactor operation allocated to experiments in the applicable facility, with abbreviations in Figure 5.1 explained in table 5.3 following. There were 648 hours of utilization for experiments; operations supported irradiation in more than one experiment facility simultaneously for 83.5 hours in 2009, therefore total time for reactor operations was 564.5 hours. The number of operating hours allocated to experiments exceeds the "console key on" time; console training includes periods the reactor key is not in the on position.

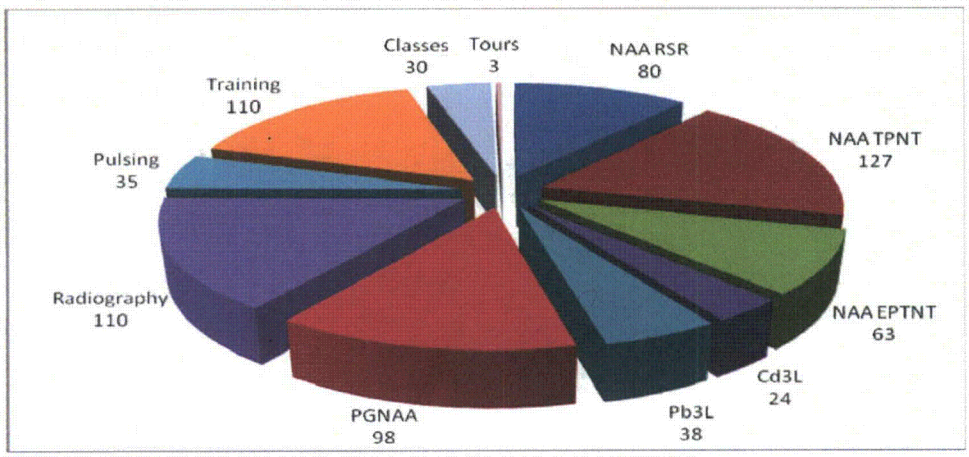

Figure 5-1, Utilization of Experiment Hours

Table 5.3

Terminology for Figure 5-1	
PGNAA	Prompt Gamma Neutron Activation Analysis
Pb3L	Sample material irradiated in the lead-lined (enhanced for lower gamma) 3-element position in-core facility (isotope production)
Cd3L	Sample material irradiated in the cadmium-lined (enhanced for epithermal neutrons) 3-element position in-core facility (generally used for NAA)
NAA EPTNT	Neutron Activation Analysis for samples irradiated in epithermal neutron pneumatic tube (irradiation position lined with cadmium)
NAA TPNT	Neutron Activation Analysis for samples irradiated in thermal neutron pneumatic tube
NAA RSR	Neutron Activation Analysis for samples irradiated in rotary specimen rack
Tour	General facility tours
Classes	Academic support (*ME337, ME361, ME388, ME389N, Health Physics, Summer Nuclear Engineering Institute*)
Training	Operations supporting reactor operator training or requalification program
Pulse	Time required to support approximately 36 pulses during 2009
Radiography	Neutron radiography

In addition to reactor facilities, the NETL has a subcritical assembly, various radioisotope sources, radiation producing machines, and laboratories for spectroscopy and radiochemistry.

5.1 Upper Grid Plate 7L and 3L Facilities

The upper grid plate of the reactor contains four removable sections configured to provide space for experiments otherwise occupied by fuel elements (two three-element and two seven-element spaces). Containers can be fabricated with appropriate shielding or neutron absorbers to tailor the gamma and neutron spectrum to meet specific needs. Special cadmium-lined facilities have been constructed that utilize three element spaces.

5.2 Central Thimble

The reactor is equipped with a central thimble for access to the point of maximum flux in the core. The central thimble is an aluminum tube extending through the central penetration of the

top and bottom grid plates. Typical experiments using the central thimble include irradiation of small samples and the exposure of materials to a collimated beam of neutrons or gamma rays.

5.3 Rotary Specimen Rack (RSR)

A rotating (motor-driven) multiple-position specimen rack located in a well in the top of the graphite reflector provides for irradiation and activation of multiple samples and/or batch production of radioisotopes. Rotation of the RSR minimizes variations in exposure related to sample position in the rack. Samples are loaded from the top of the reactor through a tube into the RSR using a specimen lifting device. A design feature provides the option of using pneumatic pressure for inserting and removing samples.

5.4 Pneumatic Tubes

A pneumatic transfer system supports applications using short-lived radioisotopes. The in-core terminus of the system is normally located in the outer ring of fuel element positions. The sample capsule is conveyed to a sender-receiver station via pressure differences in the tubing system. An optional transfer box permits the sample to be sent and received to three different sender-receiver stations. One station is in the reactor confinement, one is in a fume hood in a laboratory room, and the third operates in conjunction with an automatic sample changer and counting system.

5.5 Beam Port Facilities

Five neutron beam ports penetrate the concrete biological shield and reactor water tank at core level. Specimens may be placed inside a beam port or outside the beam port in a neutron beam from the beam port. The beam ports were designed with different characteristics to accommodate a wide variety of experiments. Shielding reduces radiation levels outside the concrete biological shield to safe values when beam ports are not in use. Beam port shielding is configured with an inner shield plug, outer shield plug, lead-filled shutter, and circular steel cover plate. A neutron beam coming from a beam port may be modified by using collimators, moderators and/or neutron filters. Collimators are used to limit beam size and beam divergence. Moderators and filters are used to change the energy distribution of neutrons in beams (e.g., cold moderator).

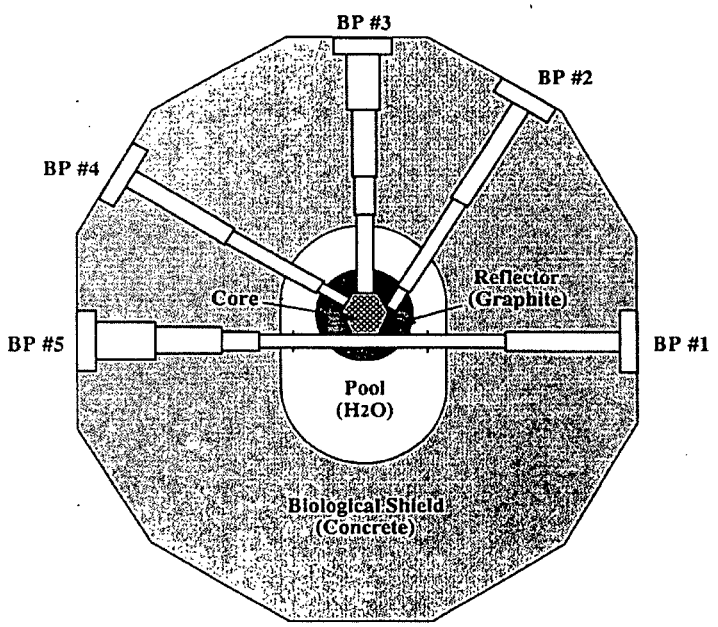

Figure 5-2, Beam Ports

Table 5-4, Dimensions of Standard Beam Ports		
BP#1, BP#2, BP#4		
At Core	6 in.	15.24 cm
At Exit	8 in.	20.32 cm
BP #3, BP#5		
At Core	6 in.	15.24 cm
8 in.	20.32 cm	
10 in.	25.40 cm	
At Exit:	16 in.	40.64 cm

5.5.1 Beam Port 1 (BP1)

BP1 is connected to BP5, forming a through port. The through port penetrates the graphite reflector tangential to the reactor core, as seen in Figure 5-2. This configuration allows introduction of specimens adjacent to the reactor core to gain access to a high neutron flux from

either side of the concrete biological shield, and can provide beams of thermal neutrons with relatively low fast-neutron and gamma-ray contamination.

A reactor-based slow positron beam facility is being fabricated at BP1. The facility (Texas Intense Positron Source) will be one of a few reactor-based slow positron beams in the world. The Texas Intense Positron Source consists of a copper source, a source transport system, a combined positron moderator/remoderator assembly, a positron beam line and a sample chamber.

The copper source will be irradiated in the middle section of the through port (BP1-BP5). The isotope ^{64}Cu formed by neutron capture in ^{63}Cu (69 % in natural copper) has a half life of 12.7 hours, with the branching ratio for β^+ emission of 19 %.. A source transport system in a 4 meter aluminum system will be used to move the source to the irradiation location and out of the biological shield. The source will be moved away from the neutron beam line outside the biological shielding to an ultra high vacuum (at around 10^{-10} torr) chamber, where the moderator assembly is located. High energy positrons from the source will be slowed down to a few eV by a tungsten foil moderator that also acts as a remoderator to reduce the beam size to enable beam transport to a target for experimentation. The beam will be electrostatically guided to deliver about 10^8 positrons/sec in the energy range of 0 - 50 keV.

5.5.2 Beam Port 2 (BP2)

BP2 is a tangential beam port, terminating at the outer edge of the reflector. A void in the graphite reflector extends the effective source of neutrons into the reflector for a thermal neutron beam with minimum fast-neutron and gamma-ray backgrounds. Tangential beams result in a "softer" (or lower average-) energy neutron beam because the beam consists of scattered reactor neutrons. BP2 is configured to support neutron depth profiling applications, with a prompt-gamma neutron activation analysis sharing the beam port.

<u>Neutron Depth Profiling (NDP)</u> Some elements produce charged particles with characteristic energy in neutron interactions. When these elements are distributed near a surface, the particle energy spectrum is modulated by the distance the particle traveled through the surface. NDP

uses this information to determine the distribution of the elements as a function of distance to the surface.

Prompt-Gamma Neutron Activation Analysis (PGNAA) Characteristic gamma radiation is produced when a neutron is absorbed in a material. PGNAA analyzes gamma radiation to identify the material and concentration in a sample. PGNAA applications include: i) determination of B and Gd concentration in biological samples which are used for Neutron Capture Therapy studies, ii) determination of H and B impurity levels in metals, alloys, and semiconductor, iii) multi-element analysis of geological, archeological, and environmental samples for determination of major components such as Al, S, K, Ca, Ti, and Fe, and minor or trace elements such as H, B, V, Mn, Co, Cd, Nd, Sm, and Gd, and iv) multi-element analysis of biological samples for the major and minor elements H, C, N, Na, P, S, Cl, and K, and trace elements like B and Cd.

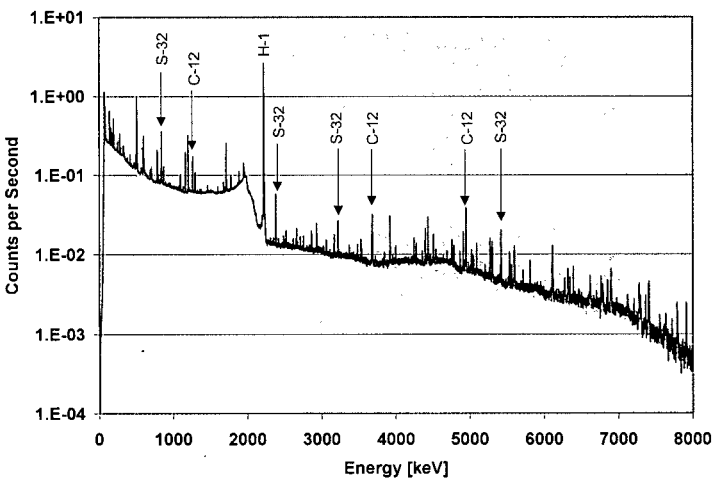

Figure 5-3, PGAA Spectra of Carbon Composite Flywheel

5.5.3 Beam Port 3 (BP3)

BP3 is a radial beam port. BP3 pierces the graphite reflector and terminates at the inner edge of the reflector. This beam port permits access to a position adjacent to the reactor core, and can provide a neutron beam with relatively high fast-neutron and gamma-ray fluxes. BP3 contains the Texas Cold Neutron Source Facility, a cold source and neutron guide system.

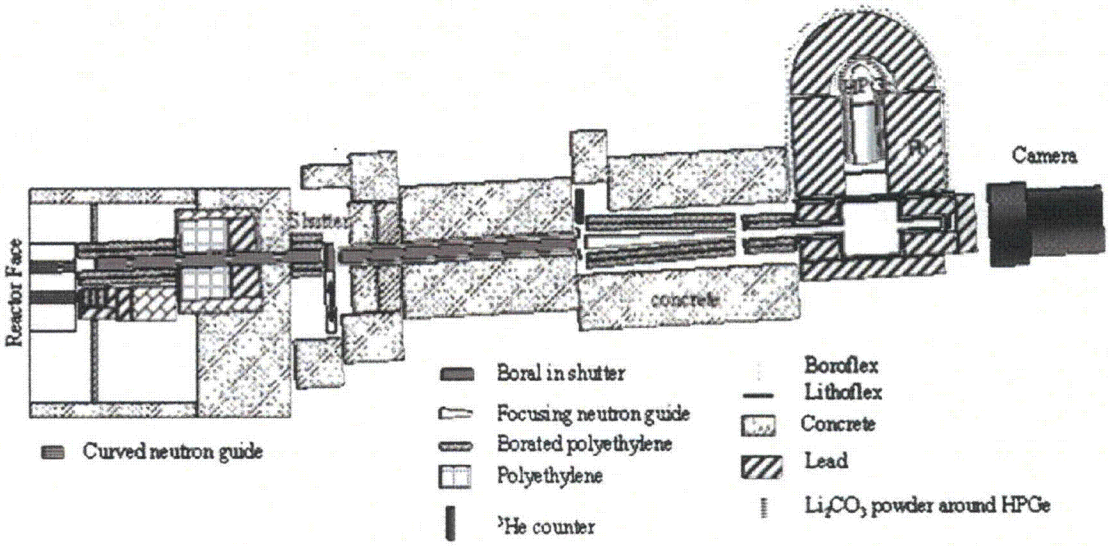

Figure 5-4, Prompt Gamma Focused-Neutron Activation Analysis Facility

Texas Cold Neutron Source. The TCNS provides a low background subthermal neutron beam for neutron reaction and scattering research. The TCNS consists of a cooled moderator, a heat pipe, a cryogenic refrigerator, a vacuum jacket, and connecting lines. The TCNS uses eighty milliliters of mesitylene moderator, maintained by the cold source system at ~36 K in a chamber within the reactor graphite reflector. A three-meter aluminum neon heat pipe, or thermosyphon, is used to cool the moderator chamber. The heat pipe working fluid evaporates at the moderator chamber and condenses at the cold head.

Cold neutrons from the moderator chamber are transported by a 2-m-long neutron guide inside the beam port to a 4-m-long neutron guide (two 2-m sections) outside the beam port. Both neutron guides have a radius of curvature equal to 300 m. All reflecting surfaces are coated with Ni-58. The guide cross-sectional areas are separated into three channels by 1-mm-thick vertical walls that block line-of-sight radiation streaming.

Prompt Gamma Focused-Neutron Activation Analysis Facility The UT-PGAA facility utilizes the focused cold-neutron beam from the Texas Cold Neutron Source. The PGAA sample is

located at the focal point of the converging guide focusing system to provide an enhanced reaction rate with lower background at the sample-detector area as compared to other facilities using filtered thermal neutron beams. The sample handling system design permits the study of a wide range of samples and quick, reproducible sample-positioning.

5.5.4 Beam Port 4 (BP4)

BP4 is a radial beam port that terminates at the outer edge of the reflector. A void in the graphite reflector extends the effective source of neutrons to the reactor core. This configuration is useful for neutron-beam experiments which require neutron energies higher than thermal energies. BP4 was configured in 2005 to support student laboratories.

5.5.5 Beam Port 5 (BP5)

A Neutron Radiography Facility is installed at BP5 (Figure 5-5). Neutrons from BP5 illuminate a sample. The intensity of the exiting neutron field varies according to absorption and scattering characteristics of the sample. A conversion material generates light proportional to the intensity of the neutron field as modified by the sample.

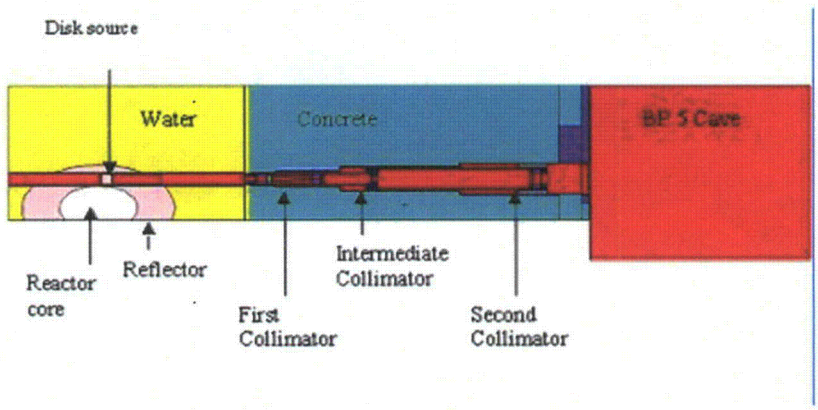

Figure 5-5, Neutron Radiography System

The conversion material is integral in one imaging system at the NETL; there are two independent conversion devices available at the NETL. A Micro-Channel Plate image

intensifying technology system (NOVA Scientific) is characterized by high resolution (up to 30 μm) over a small (approximately ½ in.) field of view. A larger image can be obtained using a more conventional 7X7 in.^{2} ^{6}LiF/ZnS scintillation screen.

A conversion screen mounted on a video tube provides a direct single in one neutron radiography camera at the NETL. The image produced by the independent conversion apparatuses can eb recorded in one of three available digital cameras. Cameras include a charge injection device (CID) camera, a cryogenically cooled charge coupled device (CCD) camera, and an electronically cooled CCD camera. The digital image is captured in a computer, where image analysis software produces the final product.

5.6 Other Experiment and Research Facilities

The NETL facility makes available several types of radiation facilities and an array of radiation detection equipment. In addition to the reactor, facilities include a subcritical assembly, various radioisotope sources, machine produced radiation fields, and a series of laboratories for spectroscopy and radiochemistry.

5.6.1 Subcritical Assembly

A subcritical assembly of 20% enriched uranium in a polyethylene moderated cylinder provides an experimental device for laboratory demonstrations of neutron multiplication and neutron flux measurements. A full critical loading of fuel previously at the Manhattan College Zero Power Reactor is currently at the facility.

5.6.2 Radioisotopes

Radioisotopes are available in a variety of quantities. Gamma and beta sources generally in microcurie to millicurie quantities are available for calibration and testing of radiation detection equipment. Neutron sources of plutonium-beryllium and californium-252 are available. Laboratories provide locations to setup radiation experiments, test instrumentation, prepare

materials for irradiation, process radioactive samples and experiment with radiochemical reactions.

5.6.3 Radiation Producing Machines

The NETL houses a 14-MeV neutron generator. The generator is currently being developed for high-energy neutron activation analysis and portable neutron radiography applications.

5.6.4 Support Laboratories

There are several laboratories adjacent to the reactor. One laboratory supports sample and standards preparation. Labs are also used for various types of radio assay, with one dedicated to a receiving station for rabbit system operations and sample counting. A control system permits automated operations.

The DOE is anticipating a loss of nuclear workforce with limited prospects for replacement of radio chemists in the national laboratory system. Therefore, a graduate-level radiochemistry laboratory was developed with support from the Department of Energy (DOE). The laboratory consists of state-of-the-art Alpha Spectroscopy Systems, Liquid Scintillation Counting System and several High Resolution Gamma Counting Systems. Students are encouraged to develop skills and interests that make them viable replacements for the nuclear workforce.

Table 5.1

Dr. Sheldon Landsberger , Professor, Nuclear and Radiation Engineering Program Manger
Hayden Head Centennial Professorship
NUCLEAR FORENSICS EDUCATION AWARD PROGRAM
Upgrade of Gamma Ray Spectroscopy Systems at the University of Texas Nuclear Engineering Teaching Lab
COMPUTATIONAL SUPPORT FOR NUCLEAR REACTOR DESIGN AND MODELING OF NEUTRON RADIOGRAPHY SYSTEMS AT SANDIA NATIONAL LAB
Historically Black Colleges and Universities Educational and Research Outreach Program in Nuclear Science and Engineering from the University of Texas at Austin
Gamma-Ray Coincidence Methods for Fission Product Identification
UTPB/UTA EDUCATION COLLABORATION
2008-2009 Nuclear Training Fellowship
UNIVERSITY RESEARCH PROGRAM IN ROBOTICS (URPR)

Dr. Steven Biegalski, Associate Professor & Director, Nuclear Engineering Teaching Laboratory

Advanced Radiological-Nuclear Detection and Forensics (HDTRA1-08-1-0032) $368,338, June 2008 to May 2010

Modeling of Radioxenon Production and Release Pathways (DE-AC52-09NA28608) $511,000, Jan. 2009 to Dec. 2011

Summer Nuclear Engineering Institute for Texas and Big-12 Undergraduates (NRC-38-08-919) $220,000, Sept. 2009 to Aug. 2010

Improving the Capability of Radioxenon Measurements by Reducing the Memory Effect (95172)$540,000, June 2009 to May 2011

Short-Lived Fission Product Yield Determinations (Proposed) $600,000, June 2010 to May 2013

Table 5.2

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Dr. Sheldon Landsberger

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Dr. Steven Biegalski

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6.0 FACILITY OPERATING SUMMARIES

6.1 Operating Experience

The UT-TRIGA reactor operated for 565 hours on 157 days in 2009, producing a total energy output of 235 MW-hrs. The history of operations over the past seventeen years of facility operation is provided in Figures 6-1 and 6-2. As illustrated, operating time has shown a marked increase from the first several years and has been relatively stable for the past decade. Varying research requirements over the past few years have led to a decrease in total energy generation.

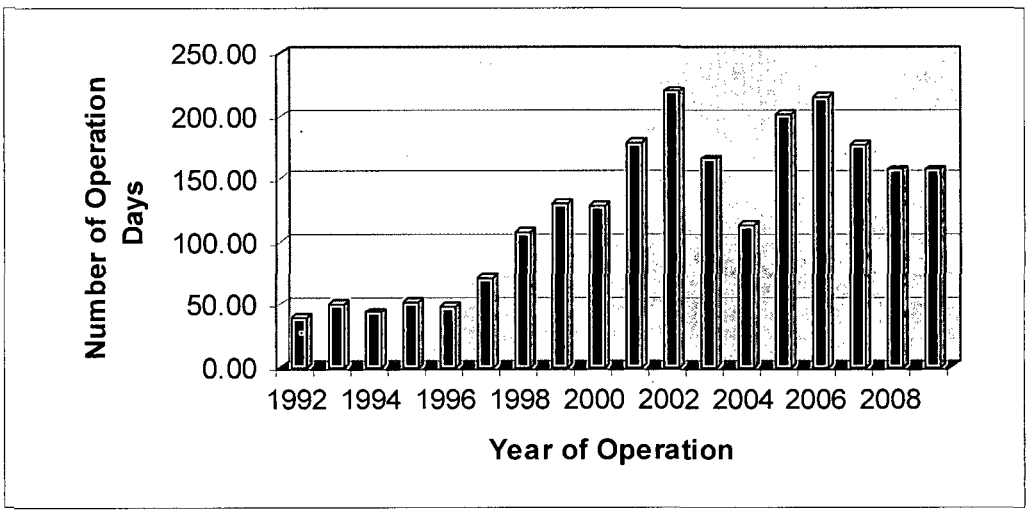


Figure 6-1, Days of Operation

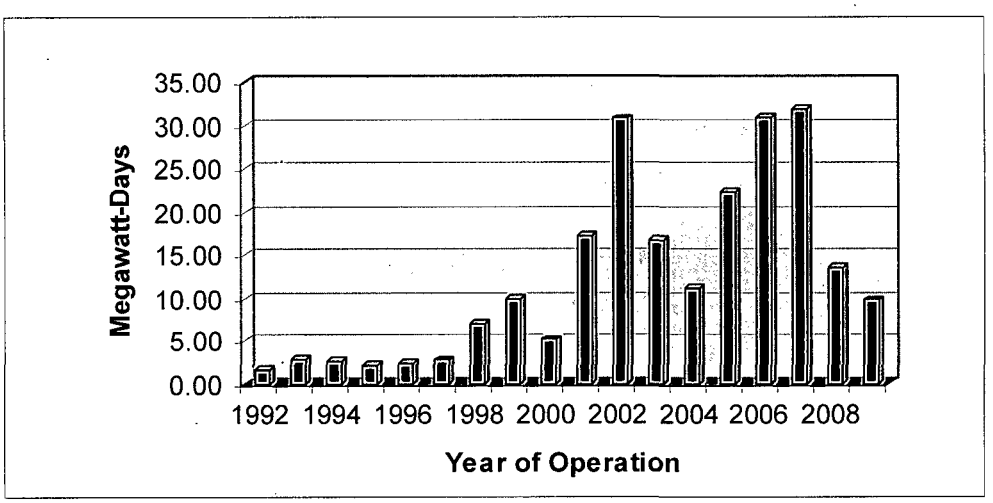


Figure 6-2 Power Production

6.2 Unscheduled Shutdowns

Reactor safety system protective actions are classified as limiting safety system (LSSS) trip, a limiting condition for operation (LCO) trip or a trip of the SCRAM manual switch. The use of the manual scram switch in normal reactor shutdowns is not a protective action. The following definitions in Table 6.1 classify the types of protective actions recorded.

Table 6.1, Protective Action Definitions

Protective Action	Description
Safety System Setting LSSS	Automatic shutdown actuated by detection of limiting safety system setting such as fuel temperature or percent power
Condition for Operation LCO - (analog detection)	Automatic shutdown actuated detection of a limiting condition for operation within a safety channel or the instrument control and safety system such as pool water level, a loss of detector high voltage or an external circuit trip
Condition for Operation LCO - (digital detection)	Automatic shutdown actuated by software action detecting inoperable conditions within a program function of the instrument control and safety system such as watchdog timers or program database errors
Manual Switch (protective action)	Manually initiated emergency shutdown

Table 6.2 lists 5 unscheduled shutdowns that occurred in 2009, all of which were initiated by the reactor safety system.

Table 6.2, SCRAM Log for 2009

Date	Time	Type	Comments
1/15/2009	15:31	SCRAM NP, NPP	Square Wave Operation Power Overshoot
02/05/2009	15:29	SCRAM NP, NPP	Square Wave Operation Power Overshoot
02/26/2009	15:39	SCRAM NP	Square Wave Operation Power Overshoot
06/08/2009	17:21	SCRAM FT1	Thermocouple Intermittent Failure
10/22/2009	11:05	SCRAM FT1	Thermocouple Intermittent Failure

Three scrams occurred during reactor operator training as a result of operations in the square wave mode. Square wave operation at power levels in excess of 750 kW are somewhat problematic because the digital power level monitoring has some intrinsic noise which can be

exacerbated during rapid signal changes, and because the required reactivity addition approaches prompt critical (while the reactivity calibration of the transient rod may vary slightly during a given operation for a number of reasons). Bounding analysis for pulsed operations indicates there is no safety issue associated with power level increases during square wave operation, although the trip function assures power level remains on-scale for instrumentation used in the square wave mode. The potential for a reactor trip during square wave operations is a consequence of the mode, and there is no obvious means to eliminate the potential for actuation of the safety system; however, the reactor staff is considering limiting square wave training operations to a nominal power level of 750 kW or less which could minimize the occurrence of such trips.

Two spurious high temperature trips actuated during 2009 from Fuel Temperature Channel 1. Fuel temperature channel 2 did not indicate a change in temperature and the scram signal cleared immediately. This trip has been previously observed on the FT1 circuit; attempts to isolate the trip to a specific component or recreate the failure have not been successful. The failure mode is conservative and acceptable until either the channel fails in a more consistent mode or the characteristics leading to the actuations can be identified.

6.3 Utilization

The University faculty and NETL staff continued to utilize the reactor in support of the University's education mission this year. Nuclear education, beam port research projects, and neutron activation analysis accounted for most of the reactor utilization during 2009. A special training program was provided to new NRC research reactor regulators. A new program, the Summer Nuclear Engineering Institute, was initiated with support from a Nuclear Regulatory Commission program. In addition to formal engineering classes, extra-curricular training supported successful examination for three reactor operator licenses in 2009, one senior reactor operator and two reactor operators. Various activation and analysis services were carried out in support of the overall UT mission and for public service. Some additional analytical service work was performed for outside agencies. Over 2200 samples were irradiated during 2009, fairly consistent with previous years of NETL operations as illustrated in Figure 6-3.

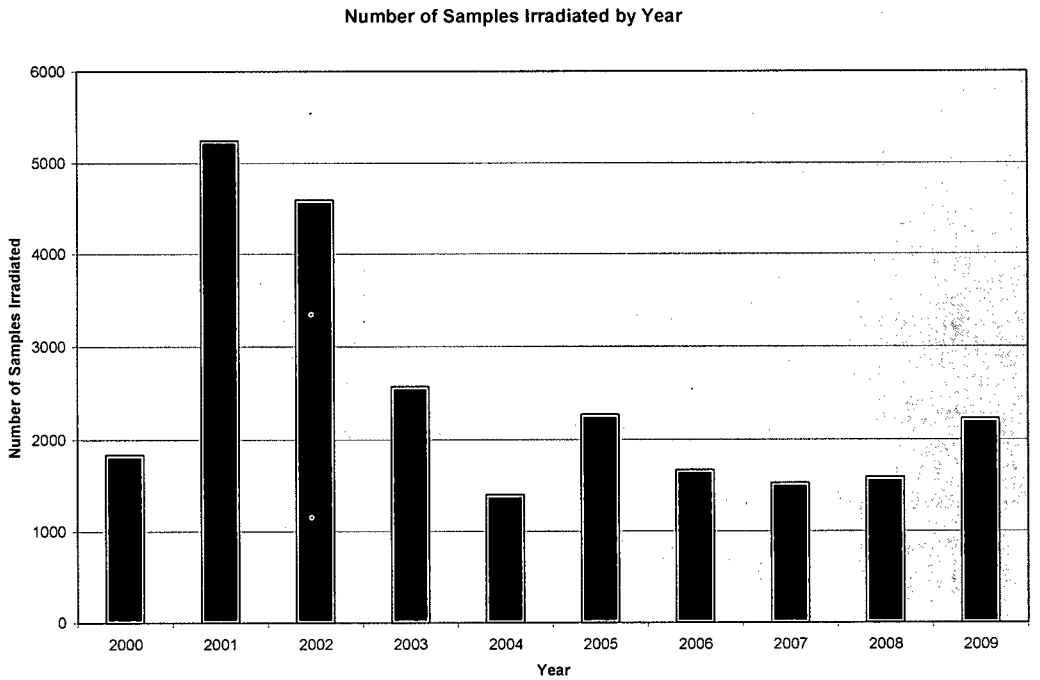


Figure 6-3, NETL Sample Activation

6.4 Routine Scheduled Maintenance

All surveillances and scheduled maintenance activities were completed during the reporting year at the required frequencies. All results met or exceeded the requirements of the Technical Specifications.

6.5 Corrective Maintenance

Activities this reporting period predominately consisted of adjustment of instrument system components (potentiometers or digital system constants) to re-tune or recalibrate the circuits to recommended levels. Corrective maintenance activities included the replacement of individual components or assemblies with like or similar replacement parts. The following list is a summary of the corrective maintenance activities:

- Replacement of burnt out indicator lamps in radiation monitoring systems.

- Replacement of carbon vain air pump impeller blades of the particulate radiation monitor.
- Replacement of lamps in control console pushbutton switches.
- Replacement of Voltage to Current Action Pac module in DAC acting as line driver to Analog % Power bar graph on control console.
- Replacement of Action Pac module in DAC used in fuel temperature test circuit.
- Adjustment of NM1000 Startup Fission Chamber cable shield ground to NEMA cabinet.
- Replacement of rod position indicator sensing potentiometer which lost mid range output linearity.
- Replacement of pool water purification system filter cartridges.
- Replacement of the Coolant System pool to chilled water differential pressure sensor.
- Replacement of HVAC and Argon Purge system fan belts.
- Replacement of reactor bay HVAC system filters.
- Replacement of personnel door weather strip.

6.6 Facility Changes

During the 2009 calendar year there were 2 changes in facility staff (Level 2, Level 3) and three new operator licenses issued by the NRC. One new experiment was authorized. Four procedure changes were accomplished, and one change to the Emergency Plan

6.6.1 Staff changes:

T. Tipping replaced D. O'Kelly as Laboratory Manager:

P. M. Whaley replaced S. O'Kelly as NETL Associate Director (previously reported)

A Senior Reactor Operator License was granted to Hunter Fuentes

Reactor operator Licenses were granted to Lee Patterson and Sam Robinson

6.6.2 Facility changes

<u>New Experiment Approval for Production of Radioactive Argon in the 3L or Center Tube experimental facilities.</u> One experiment was reviewed in 2009 under 50.59 during this reporting period. This experiment is similar to a previously approved experiment for activation of Xenon.

6.6.3 Procedure revision/updates

Four procedures were reviewed under 50.59 and implemented during this reporting period. This report provides a summary of the changes and the basis for concluding that the changes could be accomplished without prior NRC review and approval.

Control Rod Calibration. This revision incorporated information to assure quality data from measurements. This revision did not change the process, only provided additional information.

Reactor Thermal Power Calibration. This procedure was revised to accomplish the calibration at a slightly lower power level (900 versus 1000 kW) to avoid spurious safety system actuations from fluctuations in power level signal. Although nominally operations at 1000 kW are permitted, power level instrument operating characteristics leads to sporadic high power level trips when the reactor is operated at 1000 kW; therefore, normal practice is to operate at or below 950 kW. This revision did not change the process, only provided a specific power level for calibration consistent with current operating practice.

RWP procedure. The procedure was revised to provide a mechanism to permit visitors to locations where access is controlled by an RWP, and to upgrade the signature section. This change was administrative in nature, and did not change any process or requirements.

Area monitor calibrations. An exemption for calibration of a Ludlum 375 Dual Radiation Monitor was removed from the procedure; the instrument had previously been used in a research program, and was now dedicated to use as a criticality alarm under an SNM license, see following.

6.6.4 Facility Changes Accomplished in Accordance with Other Regulatory Requirements:

Facility Emergency Plan revision/update - Update of Emergency Plan to include Criticality Monitor (As part of SNM-180 License Renewal via regional NRC Office in Arlington) and update of area maps - Local review of this plan determined that this revision did not reduce the effectiveness of the plan thus a copy of the updated plan was sent to NRC RTR for information purposes as required under 10CFR 50.54.

6.7 Oversight & Inspections

Inspections of laboratory operations are conducted by university and licensing agency personnel. Two committees, a Radiation Safety Committee and a Reactor Oversight Committee review

operations of the NETL facility. The Reactor Oversight Committee convened on the dates listed in Table 6.3.

Table 6.3, Reactor Oversight Committee Reviews

First Quarter	March 2, 2009
Second Quarter	None
Third Quarter	None
Fourth Quarter	October 7, 2009

The ROC recommended that TDSHS environmental monitoring be compiled to support the renewal of the facility operating license, R-129, and that additional monitoring be considered as part of the relicense process.

Inspections by licensing agencies include federal license activities by the U. S. Nuclear Regulatory Commission (NRC), Nuclear Reactor Regulation Branch (NRR), and state license activities by the Texas Department of State Health Services (TDSHS) Radiation Control Program. An NRC inspection was conducted as indicated in Table 6.4. No findings of significance were identified.

Table 6.4, License Inspections at the NETL

License	Dates	Record
R-129	September 14-17 2009	NRC DCC Accession number ML09273002
SNM-180	None	None
L00485 (81)	None	None

Routine inspections by the Office of Environmental Health and Safety (OEHS) for compliance with university safety rules and procedures are conducted at varying intervals throughout the year. In response to safety concerns at other sites on the main campus, several additional OEHS inspections have been made. Inspections cover fire, chemical, and radiological hazards. No significant safety problems were found at NETL, which reflects favorably on the positive safety culture for all hazard classes at the NETL. Safety concerns included such items as storage of combustibles, compressed gases, and fire extinguisher access.

7.0 RADIOLOGICAL SUMMARY

7.1 Summary of Radiological Exposures

The Radiation Protection Program for the NETL facility provides monitoring for personnel radiation exposure, surveys of radiation areas and contamination areas, and measurements of radioactive effluents as indicated in Table 7.1. Site area measurements include exterior points adjacent to and distant from the building.

Table 7.1, Radiation Protection Program Requirements and Frequencies

Frequency	Radiation Protection Requirement
Weekly	Gamma survey of all Restricted Areas. Swipe survey of all Restricted Areas. Swipe survey of Radioactive Materials Areas. Response check of the continuous air monitor. Response checks of the area radiation monitors. Neutron survey of the reactor bay (during reactor operation).
Monthly	Gamma, neutron and swipe surveys of exterior walls and roof. Exchange personnel dosimeters & interior area monitoring dosimeters. Review dosimetry reports. Response check emergency locker portable rad. measuring equipment. Review Radiation Work Permits. Response check of the argon monitor. Response check hand and foot monitor. Background checks of low background alpha/beta counting system. Collect and analyze TRIGA primary water.
As Required	Process and record solid wastes and liquid effluent discharges. Prepare and record radioactive material shipments. Survey and record incoming radioactive materials. Perform and record special radiation surveys. Issue radiation work permits, provide HP for maintenance operations. Conduct orientations and training.
Quarterly	Exchange OSL environmental monitors. Gamma and swipe surveys of all non restricted areas. Swipe survey of building exterior areas. Calibrate area monitors in neutron generator room. Perform Chi-square test, and determine HV plateaus and detection efficiencies on the low background alpha/beta counting system.
Semi-Annual	Inventory emergency locker. Calibrate portable radiation monitoring instruments. Calibrate continuous air monitor, argon monitor, area rad. monitors. Calibrate personnel pocket dosimeters. Leak test and inventory sealed sources.

Annual	Conduct ALARA Committee meeting. Conduct personnel refresher training. Calibrate emergency locker portable radiation detection equipment

7.2 Summary of Radioactive Effluents

The radioactive effluent paths are ventilation for air-borne radionuclides, and the sanitary sewer system for liquid radionuclides. The most significant airborne radionuclide effluent is argon-41. Two other airborne radionuclides, nitrogen-16 and oxygen-19, decay rapidly and do not contribute to effluent releases. Argon-41, with a half-life of 109 minutes, is the only airborne radionuclide emitted by the facility during normal operations.

7.2.1 Released

There were no releases of solid radioactive materials during calendar year 2009. A small quantity of radioactive waste is stored for decay or aggregation for a shipment.

7.2.2 Discharged

Airborne Releases. A differential pressure control system in the facility assures airborne radioactive releases are controlled. The reactor room is ventilated by a general area system, and a sub-system to collect and discharge argon 41 generated from routine reactor operations. There were 5.36×10^6 μCi of argon 41 discharged during calendar year 2009, with the annual average release 2% of the value permitted by Technical Specifications.

Liquid Discharges. There are no routine releases from the facility associated with reactor operation. Large liquid-volume radioactive waste is captured in holding tanks, where liquid radioactive waste may be held for decay or processed to remove the radioactive contaminants as appropriate. To date no discharges have occurred.

Small quantities of liquid scintillation cocktail or dilute concentrations below the limits of 10 CFR 20 in the NETL laboratories may be disposed directly to the sanitary sewer. Liquid disposals are infrequent.

7.3 Radiation Exposure Received by Facility Personnel and Visitors

For calendar year 2009, no facility personnel received radiation exposures in excess of 25% of the allowed limit. Similarly, no visitors to the facility received in excess of 25% of the allowed limit.

7.4 Environmental Surveys Performed Outside the Facility

NETL monitors exterior locations indicated as positions 1 through 6 on the exterior dosimeter map. No deep dose equivalent above the minimal reporting service of 1 mrem was recorded on any of the exterior dosimeters for calendar year 2009.

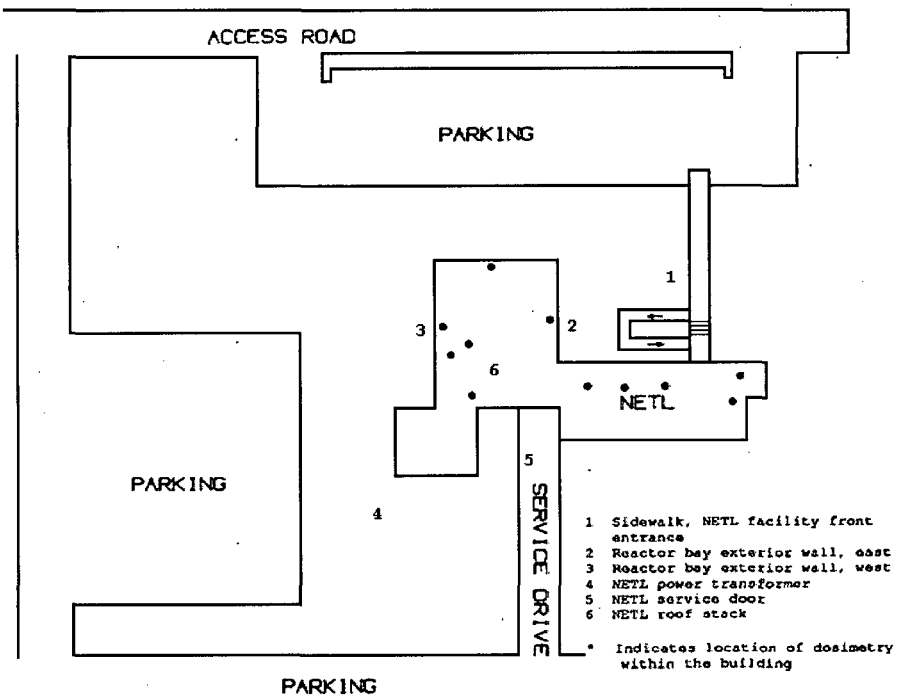
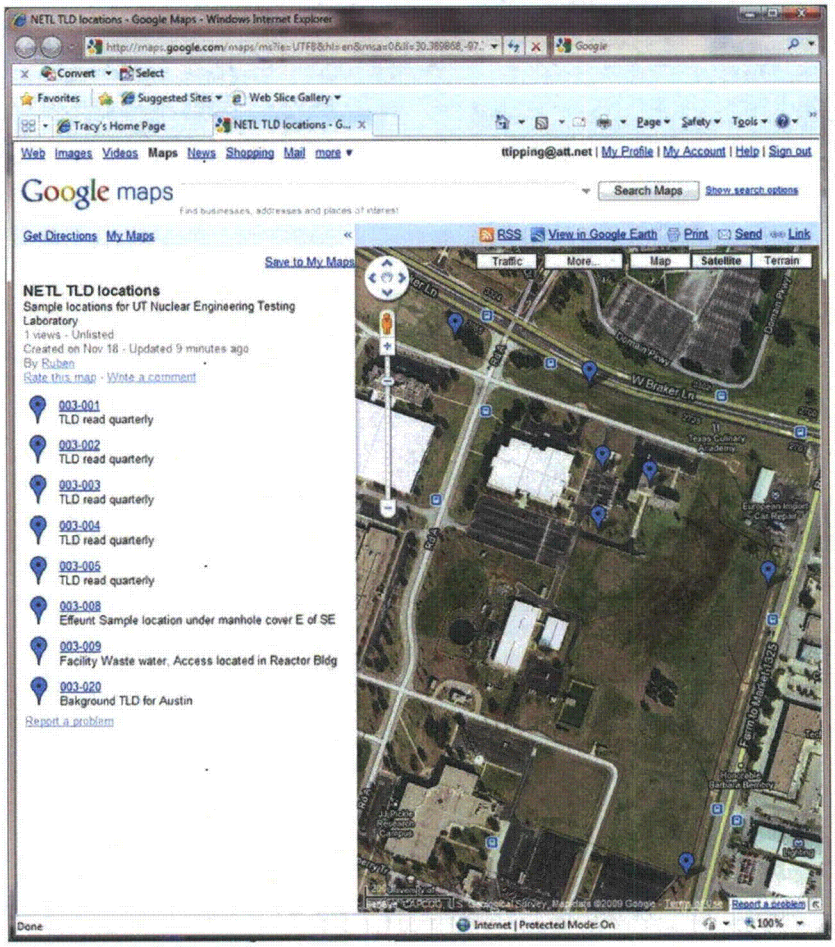


Figure 7-1, NETL Environmental Monitor Locations

In addition to the NETL monitors, the Texas Department of State Health Services monitors exterior locations near NETL indicated as positions 1 through 5 on the TDSHS TLD map. Of those five positions, the only dosimeter that indicated a dose above background was position 4 (NW of the Pickle Research Center guard post). The cumulative dose indicated at position 4 for calendar year 2009 was 5 mrem.