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March 29, 2010

U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001

ATTENTION: Document Control Desk

SUBJECT: **R.E. Ginna Nuclear Power Plant**
Docket No. 50-244

**Report of Facility Changes, Tests, and Experiments
Conducted Without Prior Commission Approval**

The subject report is hereby submitted as required by 10 CFR 50.59(d)(2). The enclosed report contains descriptions and summaries of the 10 CFR 50.59 evaluations conducted in support of proposed changes to the facility and procedures described in the UFSAR and special tests, from July 2008 through December 2009, performed under the provisions of 10 CFR 50.59. Also, for the period of July 2008 through December 2009 no regulatory commitment changes were made that met the threshold for notification to the NRC when evaluated per the guidelines of NEI-99-04, "Guidelines for Managing NRC Commitment Changes," that have not been previously submitted.

If you should have any questions regarding this submittal, please contact Tom Harding at (585) 771-5219.

Very truly yours,

Thomas L Harding

Attachment: (1) Report of Facility Changes, Tests, and Experiments Conducted Without Prior NRC Approval for July 2008 through December 2009 under the Provision of 10 CFR 50.59

cc: S.J. Collins, NRC
D.V. Pickett, NRC
Resident Inspector, NRC (Ginna)

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ATTACHMENT 1

**Report of Facility Changes, Tests, and Experiments
Conducted Without Prior NRC Approval for July 2008
through December 2009 under the Provision of 10 CFR 50.59**

Report of Facility Changes, Tests, and Experiments Conducted Without Prior NRC Approval for July 2008 through December 2009 under the Provisions of 10 CFR 50.59

50.59 Evaluation No: 2008-0002
Title of Change: Diesel Generator Service Water AOVs
Implementation Document: ECP-2008-0040
UFSAR Affected Sections: 8.3.1.1.6.2, 9.2.1.2.3, and 9.5.5
System: Service Water

Description of Change:

Two air operated valves (AOVs) in parallel, with a manual bypass valve and associated isolation valves, will be installed in each Emergency Diesel Generator (EDG) service water line to the jacket water and lube oil coolers. Solenoid valves that control the air signal to the AOVs will be energized to allow air to the actuator diaphragm to keep the associated AOV closed when the diesel generators are not running. When the solenoid is de-energized, it will change position to vent the diaphragm and the actuator spring will open the valve to provide cooling water flow to the coolers. The control of valve actuation will be integrated in the EDG start signal, such that both AOVs will open upon the EDG start signal. The AOV solenoids will be powered from a local safety related AC power distribution panel. Safety related DC power from the station batteries will be utilized to operate DC control relays and associated contacts to control the AC control power to the solenoid valves. Actuation of the Main Control Board (MCB) common alarm will occur if both AOVs fail to open within the allowed delay time set on the time delay relays.

Evaluation Summary:

Two air operated valves in parallel, with manual bypass valve and associated isolation valves will be installed in each EDG service water line to the jacket water and lube oil coolers. Associated control circuitry will also be installed. This configuration provides redundancy, since any of three 100% capacity flow paths can be aligned to the diesel generator lube oil and jacket water coolers. Implementation of this modification will change the service water cooling flow through the coolers from continuous flow, to flow initiated by the opening of the air operated valves upon a diesel generator start signal. This will minimize the deposition of debris on the cooler tube sheets, thereby reducing the differential pressure across the coolers and preserving the coolers' margin. Ultrasonic flow meter and sensors will be installed to provide indication of service water flow to the jacket water and lube oil coolers.

50.59 Evaluation No: 2009-0001

Title of Change: Implementation of Elevated RCS Li pH_t

Implementation Document: ECP-2009-0240

UFSAR Affected Sections: Table 9.3-11b

System: Reactor Core and Internals

Description of Change:

ECP-2009-0240 defines a program to raise the maximum Reactor Coolant System (RCS) lithium (Li) concentration to 6 ppm (nominal measured value) from the current limit of 3.5 ppm at full power, and up to 6.25 ppm for operation prior to achieving full power.

Evaluation Summary:

This 50.59 Evaluation assessed a program to raise the maximum RCS lithium concentration to 6 ppm (nominal measured value) from the current limit of 3.5 ppm at full power, and up to 6.25 ppm for operation prior to achieving full power. The resulting increase in RCS pH_t is expected to result in reduced corrosion of RCS materials, a reduction in fuel crud deposition, and lower plant dose rates.

Each of the eight questions in 10 CFR 50.59(c)(2)(i) through 10 CFR 50.59(c)(2)(viii) were answered "NO" such that Nuclear Regulatory Commission permission was not required to make the change.