

INSTRUCTIONAL COVER SHEET

PROGRAM	OPERATIONS TRAINING			
COURSE TITLE		JMBIA GENERATING STATION SIMULATOR		
LESSON TITLE	Swap CRD pumps; RPS-B Trip with a Scrammed Rod That Fails To Insert and FDR-V-4 that Fails to Close; RFP-B Control Oil Pressure Drop; Steam Leak Causes RCIC Isolation; RCIC-V-8 & 63 Fail to Auto Close; Feed Line Rupture; HPCS-P-1 Shaft Break; LOCA; ED On RPV Low Level			p; Steam Feed
	LENGTH OF	LESSON 1.5 Hours		
	INSTRUCT	FIONAL MATERIALS INCLUDED		
Lesson Plan PQD C	Code		Rev. No.	·
Simulator Guide PQ	QD Code	LO001677		0
JPM PQD Code			Rev. No.	
Exam PQD Code			Rev. No.	·
DIVISION TITLE	Nuclear Tra	ining		
DEPARTMENT	Operations	Training		
PREPARED BY	Ron Hayder	1	DATE _	05/28/09
REVISED BY			DATE _	
VALIDATED BY			DATE _	
TECHNICAL REV	IEW _		DATE _	
INSTRUCTIONAL	REVIEW _		DATE _	
APPROVED	_	Operations Training Manager	DATE _	
		Operations Training Manager		

Facility: Columbia	NRC Scenario No: 1
Examiners:	Operators:
	

Initial conditions: Columbia is operating in MODE 1 at full Power.

Turnover: Swap running CRD pumps to CRD-P-1A running and CRD-P-1B in stby

Event	Timeline	Event	Event Description
No.		Type*	
1.	T = 0	N (ATC)	Swap CRD Pumps from CRD-P-1A running to CRD-P-1B running
2.	T = 10	C (All) TS (CRS)	Loss of RPS-B; FDR-V-4 Fails to Close; Also - a scrammed rod. Rod 18-11 fails to fully insert
3.	T = 10	R (ATC) TS (CRS)	Reduce Reactor Power with RRC Flow to LE 80 Mlbm/hr Drive control rod 18-11 that failed to fully insert
4.	T = 25	C (ATC)	RFW B Control Oil pressure loss; Aux Oil Pump fails to auto start
5.	T = 30	C (BOP) TS (CRS)	RCIC Steam Line Break; RCIC-V-8 & RCIC-V-63 fail to auto close
6.	T = 50		Leak in the CBP discharge piping leading to a reactor scram and loss of the Condensate and Feedwater systems
7.	T = 55	M (All)	LOCA - Containment Sprays
8.	T = 55	C (BOP)	HPCS-P-1 Shaft Break
9.	T = 55		LOCA - RPV Level Drop
10.	T = 70	M (All)	Emergency Depressurization when RPV Level Drops to TAF
11.	T = 75		Re-Initiation of Containment Sprays

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

NRC Scenario No. 1

Columbia Generating Station ILC NRC Exam October, 2009

SCENARIO DESCRIPTION

The scenario begins with Columbia operating at full power. Turnover information will direct CRD Pumps be swapped.

- **EVENT 1** Swap CRD pumps to CRD-P-1B running and CRD-P-1A in standby
- **EVENT 2** After the CRD pump swap, there will be a loss of RPS B due to a failed MG set motor. FDR-V-4 fails to close on the loss of RPS and will not close when manually attempted.
- **EVENT 3** On the loss of RPS B, control rod 18-11 will scram due to blown fuses on the A RPS side. The rod will not fully insert and is at position 10. The crew will reduce core flow to LE 80 Mlbm/hr and drive rod 18-11 full in.
- The CRS will refer to Tech Specs for the scrammed control rod 18-11.
- The CRS will refer to Tech Specs for FDR-V-4.
- RPS B will be repowered from alternate power supply.
- **EVENT 4** A slow loss of control oil pressure on the B Reactor Feedwater Pump. The Auxiliary Oil Pump will fail to auto start but can be started manually.
- <u>EVENT 5</u> A steam line rupture in the RCIC supply line that causes a RCIC isolation. On the isolation, RCIC-V-8 and RCIC-V-63 fail to auto close. When manually attempted both RCIC-V-8 and RCIC-V-63 will close. The CRS will enter ABN-RAD-HIGH and EOP PPM 5.3.1, Secondary Containment Control and will refer to Tech Specs for RCIC System being OOS.
- **EVENT 6** A rupture of the condensate line downstream of the Condensate Booster Pumps that will require a manual Reactor scram. The RFW pumps will trip and all Condensate and Condensate Booster Pumps will have to be secured in order to stop the leak.
- **EVENT 7** A LOCA will develop shortly after the Reactor scram. Containment sprays will be initiated as appropriate.
- **EVENT 8** When RPV Level drops to -50 inches, HPCS will initiate and the MSIVs will close. When HPCS initiates, HPCS-P-1 shaft breaks.
- **EVENT 9** RPV level will drop to Top Of Active Fuel in 14 minutes.
- **EVENT 10** When RPV level reaches –163 inches, the CRS will direct an Emergency Depressurization be performed.
- **EVENT 11** When RPV Level is GT TAF, Containment Sprays will be initiated as appropriate.
- The scenario will be terminated when RPV level is being returned with injection from low pressure ECCS pumps, Sprays have been re-initiated as appropriate, or as directed by the scenario coordinator.

Event No. 1

Description: Swap CRD Pumps to CRD-P-1B running and CRD-P-1A in standby

The event is initiated by turnover information and will be performed after shift turnover

Time	Position	Applicants Actions or Behavior
T = 0	CRS	Directs to swap CRD pumps to CRD-P-1B running and CRD-P-1A in standby
	ATC	 Refers to SOP-CRD-Pumps Section 5.1: Step 5.1.1 - Contacts OPS 2 and verifies oil level in each CRD-P-1B oil bottle is GE ½ full Step 5.1.2 - Contacts OPS2 and verifies oil level in CRD-P-1B gear unit bulls eye GE ½ full Flags P603.A7 3.6 and 5.8 as expected alarms Step 5.1.3Places CRD-FC-600 in MANUAL Ensures announcement is made for CRD pump swap
		 Step 5.1.4 - Starts CRD-P-1B Step 5.1.5 - Stops CRD-P-1A Step 5.1.6 - Nulls CRD-FC-600 Shift CRD-FC-600 to Automatic 5.1.8 - Slowly adjusts CRD-V-3 to establish approximately 260 psid on CRD-DPI-602

ROLEPLAY: As OPS2 report CRD-P-1B running normally

NRC Scenario No. 1

Columbia Generating Station ILC NRC Exam October, 2009

Event No. 2

Description: Loss of RPS-B; FDR-V-4 Fails to Close; Control Rod 18-11 scrams but fails to fully insert (see event 3)

This event is initiated when the RCC pump swap is completed by **ACTIVATING TRIGGER 1**

Time	Position	Applicants Actions or Behavior
T = 10	ATC	Acknowledges alarms
		Reports half scram system B
		Reports control rod scram (& accumulator light) illuminated 18-11 and it is NOT full in (at position 10)
		Reports power/pressure/level
		Refers CRS to ABN-ROD due to the scrammed control rod
		(Refer to Event 3 for the remainder of actions associated with control rod)
	ВОР	Informs CRS of RPS Trouble annunciator.
		Refers to 4.800.C5 DROP 9-5 RPS BUS B TROUBLE:
		 Announces to immediately stop all maintenance and surveillance testing in progress that has a potential for generating a trip on unaffected RPS channel (RPS A system)
		Refers CRS to ABN-RPS
		Contacts OPS 2 and directs investigation of RPS B loss

ROLEPLAY: As OPS 2 wait three minutes and report that the B RPS MG Set is not running, the motor is hot to the touch, and there is an electrical smell in the room with no indication of fire

Event No. 2		
	ВОР	Per ABN-RPS:
		Throttles open RWCU-V-104
		Ensures all automatic actions have occurred
		When checked, notes that FDR-V-4 did not close
		Attempts to close FDR-V-4 and notes and reports that it did not close when C/S taken to close position
	SRO	Refers to Tech Spec 3.6.1.3 and notes condition A applies Directs RPS-B be energized from alternate power
	ВОР	Repowers RPS B as follows:
		 Verifies alternate power available by observing Alternate Feed white light illuminated
		Places the RPS Power Source Select Switch in position ALT B
DOOTH C	NDED A TO	P: When the SCRAM (NOT R RPS) is reset remove the

BOOTH OPERATOR: When the <u>SCRAM (NOT B RPS)</u> is reset, remove the scrammed rod malfunction in the Director window (right click on the line and select remove)

Event No. 2			
ВОР	Restores RPS B per step 4.6.2:		
	• Reset half scram at H13-P603		
	 Resets MS Line Monitors MS-RIS-610B and 610D at P633 		
	Depresses Isolation logic A & B reset P/B		
	 Depresses Isolation logic C & D reset P/B 		
	• Resets RC-1 by depressing WMA-RMS-FAZ/3AXY on RC-1		
	• Resets RC-2 by depressing WMA-RMS-FAZ/3BXY on RC-2		
	Opens RRC-V-19 and RRC-V-20		
	 Opens EDR-V-19 and EDR-V-20 (LEAK DET DRYWELL EQUIP DRAIN FLOW HIGH is an anticipated annunciator) 		
	• Opens TIP-V-15		
	Returns RWCU to service per SOP-RWCU-START		

COMMENTS: Event 4 may be commenced any time after RC-1 and RC-2 are reset.

NRC Scenario No. 1

Columbia Generating Station ILC NRC Exam October, 2009

Event No. 3

Description: Control Rod 18-11 Scrams but fails to go full in (stuck at position 10)

The event is initiated when the loss of RPS B is activated

Time	Position	Applicants Actions or Behavior	
T = 10	ATC	Reports control rod scram light illuminated for 18-11	
		Reports entry into ABN-ROD	
		Reports control rod 18-11 does not indicate full in	
		Selects control rod 18-11 and reports it is at position 10	
	SRO	Enters ABN-ROD and directs subsequent actions per section 4.2:	
		Directs core flow be lowered to LE 80 Mlbm/hr	
	ATC	Observes both RRC pump individual controllers are in Auto (RRC-M/A-R676A and 676B) and depresses the LOWER Pushbutton on the Master Controller (RRC-M/A-R675) to lower flow as directed	
		Reports RRC flow is LE 80 Mlbm/hr (should be about 42 Hz) (RPV High level alarm is expected)	
	SRO	Per ABN-ROD directs section 4.2.3 be performed	
	ATC	Per ABN-ROD step 4.2.3:	
		• Selects rod 18-11	
		 Depresses the Continuous Insert P/B 	
		• Drives rod 18-11 full in	
		• Releases Continuous Insert P/B and verifies it remains full in	
		Resets rod accumulator trouble and control rod drift annunciators	
		 Directs CRS to Tech Spec – Reactivity 	
		Reports power/level/pressure after rod insertion	

Event No.	Event No. 3		
	ATC	Acknowledges SDV Not Drained Alarm and refers to ARP (This alarm may or may not annunciate depending on speed of RPS B reset. It takes about 8.5 minutes to get this alarm) Verifies SDV vent and drains open when Scram is reset	
	SRO	May direct BOP to check SDV level indications	
	SRO	Refers to Tech Spec 3.1.3 Condition C Requests information on slow and inoperable control rods Directs STA to initiate a MON run Notes that a CR should be generated to document the rod problems	
precondition	oning limits ha	nform the CRS that a MON run has been performed and no thermal or eve been exceeded ere are NO inop or slow control rods	
COMMEN	VTS:		

Event No. 4

Description: Slow drop of RFW-P-1B Control Oil Pressure with failure of RFW-P-AOP/1B to start.

The event is initiated any time after the BOP operator resets RC-1 and RC-2 by **ACTIVATING TRIGGER 2** (It takes about 2 minutes to get alarm)

Time	Position	Applicants Actions or Behavior	
T = 25	ATC	Acknowledges TURB B CNTR OIL PRESS LOW annunciator and refers to ARP	
		Reports current control oil pressure (at approx 70 psig) on RFT-PI-2/1B	
	ATC	Notes that RFW-P-AOP/1B should have started per ARP but is not running and informs the CRS	
	SRO	Directs RFW-P-AOP/1B be started (RO may start the pump without direction – auto action that should have occurred but did not)	
	ВОР	Dispatches OPS 3 to investigate	
	ATC	Starts RFW-P-AOP/1B and reports the pump start to CRS	
		Reports control oil pressure returned to normal and clearing of control oil pressure low annunciator	
	SRO	Contacts Production SRO/System Engineer to investigate problem with RFW-P-AOP/1B	

ROLEPLAY: If OPS 3 is contacted report no obvious signs of a control oil leak and you will continue your investigation

CO	M	/FI	VTS.

Event No. 5

Description: RCIC Steam Line Break resulting in RCIC isolation signal with RCIC-V-8 and RCIC-V-63 failing to auto close (may be closed manually)

The event is initiated when the B RFW Aux Oil Pump is started by <u>ACTIVATING TRIGGER 3</u> (It takes 90 seconds to get alarm and 4 minutes to get the RCIC isolation signal)

Time	Position	Applicants Actions or Behavior
Critical Tas	sk is to Close RO	CIC-V-8 and RCIC-V-63 to stop steam leak
T = 30	ВОР	Acknowledges REACTOR BUILD RAD HIGH alarm (602-A5 3-1) and reports TIP Drive Area, ARM-RIS- 7, reading GT 25 mr/hr and rising Informs CRS of ABN-RAD-HIGH and Secondary Containment EOP entry condition
	SRO	Announces entry into ABN-RAD-HIGH and PPM 5.3.1 'Secondary Containment Control' Per ABN-RAD-HIGH, directs evacuation of personnel in the Reactor Building

ROLEPLAY: As OPS2 report that there is steam coming from the room above the TIP room and you are leaving the Reactor Building

BOP/ATC	Evacuates Reactor Building as directed
ВОР	Acknowledges LEAK DET RWCU/RCIC PIPE AREA TEMP HIGH alarm and investigates Leak Detection Monitors to determine temperature
	Reports temperatures as they rise above alarm points
BOP/ATC	Acknowledges alarms and informs CRS of EOP Entry conditions on RWCU Room Temperature Hi-Hi and RCIC Pipe Routing Area Temperature Hi Hi Alarms H13-P601 A2 1-1 & 2-2 and H13-P601 A3 1-8 & 2-5

	SRO	Re-enters PPM 5.3.1 as necessary
	ВОР	Reports Leak Detection Monitor point A1-5, LD-TE-24K, RWCU/RCI Room 313, is GT Max Safe value of 212°F when it occurs
		Reports Leak Detection Monitor points A2-4, LD-TE-24F, is GT Alarm but LT Max Safe value of 340°F
	BOP/ATC	Reports RCIC Trip annunciator and RCIC Steam Line valves RCIC-V-and RCIC-V-63 failed to close
	SRO	Directs RCIC-V-8 and RCIC-V-63 be manually closed
	ВОР	Obtains keys and closes RCIC-V-8 and RCIC-V-63 and reports closure to CRS
	SRO	Refers to Tech Specs 3.5.3 and performs actions for Condition A:
		• Immediately verify by admin means HPCS is operable
		Restore RCIC to operable status in 14 days
	ВОР	Reports clearing of high temperature alarms and dropping area temperatures indicating the leak is isolated
OMMEN	 NTS:	

Event No. 6

Description: Leak in the CBP discharge piping leading to a loss of the condenstate/feedwater systems as a feed source and a reactor scram

The event is initiated after actions associated with RCIC steam leak are completed by <u>ACTIVATING</u> <u>TRIGGER 4</u> (It takes abut 4 minutes to get the RFW Pump low suction pressure annunciator)

Time	Position	Applicants Actions or Behavior
T = 50	ROLEPLAY: As OPS3 report there is a water line rupture in the Turbine Building 441' elevation. It looks like water is coming from the Condensate Booster Pumps discharge line and you are getting out of the area	
	ATC	Checks pressures associated with the Condensate and Feedwater systems and observes lowering RFW pump suction pressures and informs the CRS
	SRO	Enters ABN-FLOODING
		May direct evacuation all personnel from the Turbine Building
	ВОР	Announces a evacuation of personnel in the Turbine Building if directed
	ATC	Acknowledges RFW PUMP SUCT PRESS LOW alarms (If reactor has not been manually scrammed)
	SRO	Directs CRO1 to insert a manual reactor scram
	ATC	Performs immediate scram actions:
		Announces to listen up for the scram report
		Places Mode Switch to Shutdown
		Reports APRMs downscale
		Reports Reactor Pressure and trend
		Reports RPV level and trend
		Reports EOP entry condition on Low RPV Level

Event No.	Event No. 6		
	SRO	Repeats back scram report and enters PPM 5.1.1, RPV Control, on Low RPV Level	
	ATC	Reports trip of both RFW pumps on low suction pressure	
	SRO	Directs all Condensate and Condensate Booster Pumps be stopped	
	BOP/ATC	Secures Condensate and Condensate Booster Pumps as directed	
ROLEPLA rupture.	AY: If Condensa	ate pumps are not secured, as OPS3 report water is still flowing from line	
ROLEPLA	AY: If asked, as	OPS3 report the water has stopped coming from piping rupture.	
COMMEN	COMMENTS:		

Event No. 7

Description: LOCA (Containment Spray)

This event is auto initiated when the MSIVs close

Time	Position	Applicants Actions or Behavior
T = 55	ВОР	Recognizes MSIV closure and updates Crew that the MSIVs are closed and he has pressure control on SRVs at 800 to 1000 psig pressure band
	ВОР	Recognizes rising DW Pressure and reports EOP entry at 1.68 psig
		Also reports additional primary containment EOP entries as they occur
	SRO	Enters EOP 5.2.1, Primary Containment Control and re-enters 5.1.1 RPV Control, due to high DW pressure
		Directs actuations for +13", -50" and 1.68 psig be verified as appropriate
	ВОР	Reports Wetwell pressure when it reaches 2 psig
	SRO	Directs Wetwell sprays/Suppression Pool Cooling with RHR A
		Directs securing sprays if pressure drops below 1.68 psig
	ВОР	Using quick card, initiates wetwell sprays and supplements with suppression cooling
	RO/BOP	Reports Wetwell pressure when it reaches 12 psig

Event No. 7		
	SRO	Directs Drywell Cooling Fans be secured
	ВОР	Secures Drywell Cooling Fans as directed
	SRO	Directs DSIL verification Directs Drywell Sprays be initiated with RHR B Directs securing sprays if drywell pressure drops below 1.68 psig
	ВОР	Verifies within DSIL and using quick card, initiates Drywell Sprays as directed Reports Sprays effective as Drywell pressure drops

COMMENTS: Drywell Spray initiation may be delayed until after Emergency Depressurization

Event No. 8

Description: HPCS-P-1 Shaft Breaks

This event is activated at the beginning of the scenario but only realized when HPCS initiates

Time	Position	Applicants Actions or Behavior
T = 55	ВОР	When HPCS initiates on High Drywell Pressure or RPV/L at -50", verifies HPCS status Recognizes that HPCS-P-1 is running with no flow and only 100 psig discharge pressure and informs CRS
	SRO	Directs HPCS-P-1 be checked

ROLEPLAY: If asked, as OPS2 report that there are parts to the shaft coupling for HPCS-P-1 all over the HPCS Pump room

SRO	May directs HPCS-P-1 be secured
ВОР	Secures HPCS-P-1 if directed

Event No. 9

Description: LOCA (RPV Level Drop)

This event starts when scram is initiated

Time	Position	Applicants Actions or Behavior
T = 55	ATC	Reports RPV level drop
		Gives RPV level reports as level continues to lower May direct ABN-CRD-MAXFLOW be performed

NOTE – From scram time it takes about 2 minutes to get to -50"

It takes another 4 minutes to get to -129"

It takes another 90 seconds to get to TAF at -161"

SRO	Directs ADS be inhibited when ADS timers initiate (at -129")
	As level drops, expands RPV band given
ATC/BOP	When RPV/L drops to -129" and the ADS timers intimate, takes both ADS inhibit switches to inhibit
ATC/BOP	Reports RPV level as it transitions from Wide Range to Fuel Zone indicators
	Reports RPV level at -161" (TAF)

Event No. 10

Description: Emergency Depressurization and Return RPV Level to +13" to +54"

This event is initiated when RPV Level reaches -161" (Top Of Active Fuel)

Time	Position	Applicants Actions or Behavior	
Critical Tas	Critical Task is to Emergency Depressurize the RPV when RPV Level reaches TAF at -161"		
T = 70	SRO	Determines that Emergency Depressurization is required at -161"	
		Exits the pressure leg of PPM 5.1.1 and enters PPM 5.1.3, Emergency RPV Depressurization	
		Determines wetwell level is GT 17' and directs 7 ADS SRVs be opened	
	ATC/BOP	Opens 7 ADS SRVs as directed	
	SRO	Directs containment sprays and Suppression Pool cooling be secured to facilitate RPV injection	
		Directs RPV Level be returned to +13" to +54" level band	
	RO/BOP	Secures sprays and Suppression Pool cooling as directed	
		Allows ECCS injection valves to open at 470 psig and RPV injection as it occurs	
		Reports RPV level rising and when GT –161 inches	
COMMEN'	TS·		

Event No. 11

Description: Re-initiation of Wetwell and Drywell Sprays

This event is initiated by the SRO when RPV level is greater than TAF

Time	Position	Applicants Actions or Behavior
T = 75	SRO	Directs initiation of Wetwell sprays if WW Pressure is GT 2 psig
		Directs initiation of Drywell sprays if Wetwell Pressure is GT 12 psig
		Directs Suppression Pool Cooling as required
	ATC/BOP	Secures injection systems as necessary to return RPV level to +13" to +54" band
		Initiates Wetwell and Drywell sprays as appropriate
		Initiates suppression pool cooling if directed

Termination Criteria: The scenario will be terminated when RPV level is being returned with injection from low pressure ECCS pumps, Sprays have been reinitiated as appropriate, or as directed by the scenario coordinator

TURNOVER INFORMATION

Initial conditions: Columbia is operating in MODE 1 at full Power.

Turnover: A PM is scheduled to swap CRD Pumps. After shift turnover swap CRD

pumps to CRD-P-1B running and CRD-P-1A in standby.

OPS 2 is standing by for the CRD Pump swap evolution.

SIMULATOR SETUP INSTRUCTIONS

Reset to IC-15.

Put Scenario #1 schedule and Scenario #1 Event in appropriate folders on M drive

Load the schedule.

Go to run.

If using scenario for more that this time, snap into an IC.

NRC Scenario No. 1

Columbia Generating Station ILC NRC Exam October, 2009

SCHEDULE

```
<!-- This file contains a Thunder Simulations Schedule -->
<SCHEDULE>
      <ITEM row = 1>
            <TIME>1</TIME>
            <ACTION>Insert malfunction AOV-SCN013F after 1 to FAIL AS IS</ACTION>
            <DESCRIPTION>FDR-V-4 DW FLOOR DRN OUTBD ISOL</DESCRIPTION>
      </ITEM>
      <ITEM row = 2>
            <TIME>1</TIME>
            <ACTION>Insert malfunction PMP-CSS001B after 1</ACTION>
            <DESCRIPTION>HPCS-P-1 HPCS PUMP SHAFT BREAK</DESCRIPTION>
      </ITEM>
      <ITEM row = 3>
            <TIME>1</TIME>
            <ACTION>Insert malfunction MAL-RMC005-1811 after 1</ACTION>
            <DESCRIPTION>ROD 1811 STUCK at position 10</DESCRIPTION>
      </ITEM>
      <ITEM row = 4>
            <TIME>1</TIME>
            <ACTION>Insert malfunction MOV-RCI016F to FAIL AUTO CLOSE</ACTION>
            <DESCRIPTION>RCIC-V-8 Fails to Auto Close/DESCRIPTION>
      </ITEM>
      <ITEM row = 5>
            <TIME>1</TIME>
            <ACTION>Insert malfunction MOV-RCI012F to FAIL AUTO CLOSE</ACTION>
            <DESCRIPTION>RCIC-V-63 Fails to Auto Close/DESCRIPTION>
      </ITEM>
      <ITEM row = 6>
            <TIME>1</TIME>
            <ACTION>Insert malfunction ANN-820B2F06 to OFF</ACTION>
            <DESCRIPTION>GLAND SEAL STM PRESS HIGH</DESCRIPTION>
      </ITEM>
      <ITEM row = 8>
            <TIME>1</TIME>
            <EVENT>1</EVENT>
            <ACTION>Insert remote LOA-EPS277 to TRIP on event 1</ACTION>
            <DESCRIPTION>RPS-CB-MG2 MG-B OUTPT BKR O/C</DESCRIPTION>
      </ITEM>
      <ITEM row = 9>
            <TIME>1</TIME>
            <EVENT>1</EVENT>
            <ACTION>Insert malfunction MAL-RMC007-1811 on event 1</ACTION>
            <DESCRIPTION>ROD 1811 SINGLE ROD SCRAM</DESCRIPTION>
      </ITEM>
      <ITEM row = 11>
            <TIME>1</TIME>
            <EVENT>29</EVENT>
```

NRC Scenario No. 1

Columbia Generating Station ILC NRC Exam October, 2009

```
<ACTION>Insert malfunction MAL-RMC005-1811 after 1 on event 29 delete in
3</ACTION>
            <DESCRIPTION>Deletes stuck rod malfunction/DESCRIPTION>
      </ITEM>
      <ITEM row = 12>
            <TIME>1</TIME>
            <action>Event Events/L0001677.evt</action>
            <DESCRIPTION>Brings in Events/DESCRIPTION>
      </ITEM>
      <ITEM row = 14>
            <TIME>1</TIME>
            <EVENT>2</EVENT>
            <ACTION>Insert override IND-FPT022 to 68 in 120 on event 2</ACTION>
            <DESCRIPTION>RFT-PI2-1B TURBINE OIL CONTROL PRESS METER SIGNAL
M</DESCRIPTION>
      </ITEM>
      <ITEM row = 15>
            <TIME>1</TIME>
            <EVENT>2</EVENT>
            <ACTION>Insert override IND-FPT023 to 18 in 120 on event 2</ACTION>
            <DESCRIPTION>RFT-PI3-1B TURBINE OIL BEARING PRESS METER SIGNAL
M</DESCRIPTION>
      </ITEM>
      <ITEM row = 16>
            <TIME>1</TIME>
            <EVENT>2</EVENT>
            <ACTION>Insert malfunction ANN-840A1E05 after 110 to ON on event 2</ACTION>
            <DESCRIPTION>CONTROL OIL TURB B PRESS LOW</DESCRIPTION>
      </ITEM>
      <ITEM row = 18>
            <TIME>2</TIME>
            <EVENT>28</EVENT>
            <ACTION>Insert malfunction ANN-840A1E05 to ON after 1 on event 28 delete in
1</ACTION>
            <DESCRIPTION>Removes Control Oil TURB B press low on Aux Pump red light
illuminated</DESCRIPTION>
      </ITEM>
      <ITEM row = 20>
            <TIME>2</TIME>
            <EVENT>28</EVENT>
            <ACTION>Insert override IND-FPT023 to 18 after 1 on event 28 delete in
1</ACTION>
            <DESCRIPTION>Returns RFT-PI3-1B to normal on LO Pump red light
illuminated</DESCRIPTION>
      </ITEM>
      <ITEM row = 21>
            <TIME>2</TIME>
            <EVENT>28</EVENT>
            <ACTION>Insert override IND-FPT022 to 68 after 1 on event 28 delete in
            <DESCRIPTION>Returns RFT-PI2-1B to normal on LO Pump red light
illuminated</DESCRIPTION>
      </ITEM>
```

</SCHEDULE>

Columbia Generating Station ILC NRC Exam October, 2009

```
<ITEM row = 22>
            <TIME>1</TIME>
            <EVENT>3</EVENT>
            <ACTION>Insert malfunction MAL-RCI006 to 2000000 in 3600 on event
3</ACTION>
            <DESCRIPTION>RCIC BREAK BETWEENRCIC-V-8 & PCN/DESCRIPTION>
      </ITEM>
      <ITEM row = 24>
            <TIME>1</TIME>
            <EVENT>4</EVENT>
            <ACTION>Insert malfunction MAL-CFW006 to 20000 in 3600 on event 4</ACTION>
            <DESCRIPTION>LEAK IN COMMON CBP DISCHARGE</DESCRIPTION>
      </ITEM>
      <ITEM row = 26>
            <TIME>4</TIME>
            <EVENT>26</EVENT>
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NRC Scenario No. 1

Columbia Generating Station ILC NRC Exam October, 2009

EVENTS

<!-- This file contains a Thunder Simulations Event --> <EVENT>

<TRIGGER id="29" description="Deletes Stuck rod when Insert P/B light
illuminates">XRLO033I >0</TRIGGER>

</EVENT>

Appendix D

FORM ES-D-2

NRC Scenario No. 1 Columbia Generating Station ILC NRC Exam October, 2009



INSTRUCTIONAL COVER SHEET

PROGRAM TITLE	E OPERATIONS TRAINING			
COURSE TITLE	CO	LUMBIA GENERATING STATION SIMULAT	OR EXAMI	NATION
LESSON TITLE	Start ASD Channel 1A1; Raise Power with Flow; Swap RCC Pumps; 'B' Flow Unit Failure; FPC-P-1B Failure; Main Turbine Trip on High MSR Level; Hydraulic ATWS; Reduced SLC Capacity; Lower RPV Level; S/R/S Inserts Rods			gh MSR
	LENGTH O	F LESSON 1.5 Hours		
	INSTRU	CTIONAL MATERIALS INCLUDED		
Lesson Plan PQD C	ode		Rev. No.	·
Simulator Guide PQ	D Code	LO001678	Rev. No.	0
JPM PQD Code			Rev. No.	
Exam PQD Code			Rev. No.	
DIVISION TITLE DEPARTMENT	Nuclear Tra			
PREPARED BY	Ron Hayde	n	_ DATE _	05/30/09
REVISED BY			_ DATE _	
VALIDATED BY			_ DATE _	
TECHNICAL REV	IEW _		_ DATE _	
INSTRUCTIONAL	REVIEW _		_ DATE _	
APPROVED			DATE	

Appendix D	NRC Scenario No. 1 Columbia Generating Station ILC NRC Exam October, 2009	FORM ES-D-2
	Operations Training Manager	

Verify materials current IAW SWP-TQS-01 prior to use.

Facility: Columbia	NRC Scenario No: 2
Examiners:	Operators:
	

Initial conditions: Columbia is operating at reduced power. Yesterday ASD Channel 1A 1 tripped. The problem has been corrected and Channel 1A1 is ready to be re-started.

Turnover: Start ASD Channel 1A1 per SOP-RRC-ASD. Step 5.6.1 is complete. After the channel is started, raise Reactor Power with flow to exit the OPRM Enabled Region. The SNE is working on a reactivity plan to continue the power increase from that point. Also RCC Pumps need to be swapped for run time equalization. Start RCC-P-1A and place RCC-P-1C in standby.

Event Timeline Event **Event Description** Type* No. T = 0N (ATC) Start ASD Channel 1A1, RRC ASD to Auto 1. 2. T = 0N (BOP) Swap RCC Pumps to RCC-P-1A running and RCC-P-1C in Standby 3. T = 10R (ATC) Raise Power with Flow to Exit OPRM Enabled Region 4. T = 20C (BOP) RCIC Coupling bolts failed, Trip RCIC; Protect HPCS and ADS TS (SRO) TS 3.5.3A T = 305. C (ATC) Flow Unit B Failure: TS (SRO) Tech Spec (LCS 1.3.2.1 and TS 3.3.1.1) 6. T = 35(C) BOP FPC-P-1B Failure; FPC-P-1A Fails to Auto Start 7. T = 40(C) BOP MSR Drain Tank Valves Fail Causes High MSR Level and MT Trip 8. T = 45M (All) Hydraulic ATWS; Lower RPV Level and Establish LL 9. T = 50C (ATC) Reduced SLC T = 5510. N (BOP) Perform PPM 5.5.10 and 5.5.11 to insert Control Rods T = 6511. ATWS Clears and RPV Level is Returned to Normal

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

NRC Scenario No. 2

Columbia Generating Station ILC NRC Exam October, 2009

SCENARIO DESCRIPTION

The scenario begins with Columbia at approximately 56% power with RRC Pumps running at 32 Hz. ASD Channel 1A1 tripped yesterday and is now ready to be returned to service. When the Crew takes the shift they will re-start ASD Channel 1A1 and swap RCC Pumps.

- **EVENT 1** Start RRC ASD Channel 1A1
- **EVENT 2** Swap running RCC pumps to RCC-P-1A running and RCC-P-1C in standby
- **EVENT 3** When Channel 1A1 is restarted, place RRC-A ASD into Auto and raise power with flow to exit the OPRM Enabled Region.
- **EVENT 4** Failure of RCIC Coupling Bolts. The Control Room will be contacted and informed that the coupling for the RCIC Turbine to Pump has only 2 of 5 bolts engaged. The crew should insert a RCIC trip to prevent it from starting and refer to TS 3.5.3 Condition A.
- **EVENT 5** Failure of the B Flow Unit. The ATC operator will investigate alarms and determine the B Flow Unit has an active INOP trip. The CRS will refer to Tech Specs (LCS1.3.2.1 and TS 3.3.1.1) and direct the Flow Unit be bypassed.
- **EVENT 6** Failure of FPC-P-1B. FPC-P-1A will not auto start on low pressure but the BOP operator will start FPC-P-1A manually and secure FPC-P-1B.
- **EVENT 7** High Level MSR Drain Tank. A failure of the valves for MSR Drain Tank will occur. The BOP operator will attempt to take manual control and reduce drain tank level but will not be successful Eventually a MSR High Level alarm annunciates. The crew should take action and insert a manual scram and trip the Main Turbine which will trip on High MSR Level in 30 seconds from receipt of the alarm.
- **EVENT 8** Hydraulic ATWS and Lower RPV Level. When a scram is inserted the crew will determine a hydraulic ATWS occurs. Reactor Power will be about 25%. The RRC Pumps trip when the MT Trips. The CRS will direct all injection be secured and lower RPV level to LT -65". When RPV level is about -100", Reactor Power will be LT 5%
- **EVENT 9** Reduce SLC capacity. When the CRS directs SLC initiation, both SLC pumps start but only 20 gpm SLC flow will develop.
- **EVENT 10** Insert Control Rods. Rod Insertion. PPM 5.5.10 and 5.5.11 will be performed to insert Control Rods.
- **EVENT 11** The rods will insert on the first S/R/S. When all rods are in, RPV Level will be returned to +13" +54" band.

The scenario will be terminated when RPV level is being returned to normal band or as directed by the scenario coordinator.

Event No. 1

Description: Start RRC-P-1A ASD Drive 1A1

This event is initiated by shift turnover

Time	Position	Applicants Actions or Behavior
T = 0	SRO	Directs that ASD Channel 1A1 be started per SOP-RRC-ASD Section 5.6.
	ATC	Commences Section 5.6 at step 5.6.2: a. Verifies frequency of drive 1A2 at LE 35 Hz
		b. Depresses the ASD START button
		c. Verifies the red lamp illuminates (observes red light illuminates, green light goes out and channel failure limit alarm clears)
		d. Verifies heater breaker is OFF by contacting OPS3
		Informs CRS that ASD Channel 1A1 has been started

ROLEPLAY: If asked the Heater Breaker is in the OFF position

CC	7(ΛN	Æ	NΊ	rs:

Event No. 2

Description: Swap RCC Pumps to RCC-P-1A running and RCC-P-1C secured.

The event is initiated by turnover information and will be performed immediately after shift turnover

Time	Position	Applicants Actions or Behavior
T = 0	CRS	Direct RCC-P-1A be started and RCC-P-1C be secured
	ВОР	Refers to SOP-RCC-OPS Section 5.1.1a and starts RCC-P-1A: • Contacts OPS 2 and verifies RCC-V-1A is OPEN
		Contacts OPS 2 and verifies RCC-V-2A is OPEN
		 Place RCC-RMS-P-1A in AUTO after START Refers to section 5.1.2c:
		Places RCC-RMS-P-1C in AUTO after STOP
		Contacts OPS 2 and verifies RCC-V-23C CLOSED

ROLEPLAY: RCC-V-1A and RCC-V-2A are open and RCC-V-23C did close when asked

Event No. 3

Description: Place ASD in Auto and Raise Power With Flow to Exit OPRM Enabled Region

The event is initiated by the SRO when ASD Channel 1A1 has been started

Time	Position	Applicants Actions or Behavior
T = 10	SRO	Directs ATC to place A RRC Pump ASD control in Automatic
	ATC	Verifies Master and Individual controller setpoints matched and places RRC-A ASD in Auto
	SRO	Directs ATC to raise Reactor Power with flow to exit the OPRM Enabled Region at a rate not to exceed 10 MWE/minute
	ATC	Raises reactor power with flow as directed

Event No. 4

Description: RCIC Turbine Shaft Coupling Bolts Missing – Inops RCIC

The event is initiated by OPS 2 contacting the Control Room when power has been raised and the OPRM Region has been exited

Time	Position	Applicants Actions or Behavior	
T = 20	ROLEPLAY: As OPS 2 call the control room on extension 2393 and inform the reactor operator that you and the SSS are down in the RCIC Room. I found a nut on the turbine by the coupling so I called OPS 1. We investigated further and found that only two of the five bolts for the coupling that connects the turbine to the pump are still engaged.		
	ВОР	Answers the phone and informs the CRS that the RCIC turbine/pump coupling only has 2 of the 5 bolts still connected.	
	SRO	Calls Production/Work Control and informs them of the RCIC turbine situation	
	Y: If called as P together to inve	Production/Work Control acknowledge the info and tell them a team stigate.	
	SRO	Directs BOP operator to insert a manual trip of the RCIC turbine to prevent it from starting	
	ВОР	Trips the RCIC turbine by either depressing the Turbine Trip P/B or manually closing RCIC-V-1	
	SRO	Refers to Tech Specs 3.5.3 Condition A:	
		Verifies HPCS operable by administrative means and	
		Restore RCIC to operable status within 14 days	
	SRO/BOP	Refers to OI-49 and determines that HPCS and ADS-SYS 1A/B be protected	

Event No. 4	ļ	
	ВОР	Places protected signs of HPCS and both ADS divisions
		Contacts OPS2 to hang protected signs on HPCS system
COMMEN	TS:	

Event No. 5

Description: Flow Unit B Failure

The event is initiated when the OPRM Region has been exited and is initiated by **ACTIVATING**

TRIGGER 1

Time	Position	Applicants Actions or Behavior
T = 30	ATC	Acknowledges Rod Out Block and Flow Reference Off Normal annunciators, informs CRS and refers to ARP for Flow Reference Off Normal (P603 A8 3-6)
	ATC	Per ARP determines the white light is illuminated indicating Flow Unit B is INOP Refers CRS to LCS 1.3.2.1 and TS 3.3.1.1 and TS Bases page 3.3.1.1-9
	SRO	Refers to LCS 1.3.2.1 and TS 3.3.1.1 and TS Bases page 3.3.1.1-9 and determines requirement is to place in trip within 12 hours
	SRO	Directs ATC Operator to bypass Flow Unit B
	ATC	Places Flow Unit B in the Bypass position Notes the INOP light goes out and the BYPASS light illuminates

Event No. 6

Description Lowering FPC System pressure with a failure of the standby pump to Auto Start

The event is initiated when Tech Specs have been referenced for RCIC and systems protected by

ACTIVATING TRIGGER 2

Time	Position	Applicants Actions or Behavior
T = 35	ВОР	Acknowledges CIRCULATION PUMP B DISCHARGE PRESS LOW on P627.FPC2 3-1 and refers to ARP
		Informs CRS of the alarm and that the automatic actions should be an Auto start of FPC-P-1A but that FPC-P-1A did not auto start
		(The BOP may start pump and then inform CRS of actions based on auto action that should have occurred but did not)
	SRO	Directs the start of FPC-P-1A if not already running
	SRO	Directs that FPC-P-1B be secured
	ВОР	Secures FPC-P-1B as directed
	SRO	Contacts production/Work Control/OPS2 to investigate FPC-P-1B

Event No. 7

Description: Failure of MSR Drain tank 1A level control valves HD-LIC-9A and 9A2 closed

The event is initiated by <u>ACTIVATING TRIGGER 3</u> after actions for Flow Unit and FPC failures have been completed

(It takes about 6 minutes to get MSR A High Level Alarm after trigger activation)

Time	Position	Applicants Actions or Behavior		
T = 40	ВОР	Acknowledges alarm and reports MSR Drain Tank 1A Level High alarm. Refers to ARP.		
		Investigates controllers to ensure they are in Auto and recognizes the controllers are opening drain valves but level continues to rise. May attempts manual operation but valve is already full open.		
		Reports controllers functioning in Auto (or Manual) and level indication is off scale high.		
	SRO	Conduct a brief on actions if level continues to rise and MSR Level High alarm is received.		
	ATC/BOP	Acknowledges and reports MSR A Level High alarm and refers to ARP (Turbine Trip has a 30 second time delay)		
	SRO	Directs a manual reactor scram prior to automatic scram actuation (SRO may direct a scram prior to MSR A high Level alarm)		
	ATC	Scrams the reactor and performs immediate operator actions of PPM 3.3.1:		
		mode switch to shutdown		
		• monitors/reports Power/Pressure/Level		
		Recognizes failure to scram (ATWS) – (REFER TO EVENT 7)		

COMMENTS:

Event No. 8

Description: Hydraulic ATWS

This event is setup at the beginning of the scenario and occurs automatically when a manual scram is inserted

Critical Task is to lower RPV level and establish an LL (Lowered Level). **Position** Time **Applicants Actions or Behavior** T = 45ATC Continues with immediate scram actions after recognizing all control rods did not insert: Depress the manual scram pushbuttons Initiate ARI and verifies valves opened Insert SRMs and IRMs Announce EOP entry into PPM 5.1.1 on low Reactor Water level and/or Power GT 5% and a scram required Reports reactor power **SRO** Enters PPM 5.1.1 and directs/verifies that the Mode Switch has been placed in SHUTDOWN and exits PPM 5.1.1 via the Power leg to PPM 5.1.2, RPV Control ATWS Directs BOP to: Inhibit ADS and take manual control of HPCS Verify all appropriate isolations and initiations have occurred Verify pressure is being maintained by the bypass valves **BOP** Takes both ADS control switches to the INHIBIT position and

Columbia Generating Station ILC NRC Exam October, 2009

Event No. 8	
	acknowledges associated BISIs
	Arms and Depresses the HPCS system initiation P/B while holding the control switch for HPCS-P-1 to STOP
	Closes HPCS-V-4 when it get fully opened
	Reports completion to SRO
SRO	Directs bypassing the MSIV isolation interlocks per PPM 5.5.6
	Directs performance of PPM 5.5.1
ВОР	Performs PPM 5.5.6 and updates Crew
	Performs PPM 5.5.1 and updates Crew
SRO	Direct the ATC to:
	Stop and prevent all injection into the RPV except by Boron injection systems, RCIC, and CRD
	Lower level to a band less than -65 inches but greater than -183 inches (preferred band is -80" to -140").
	Records the upper limit as LL.
	Maintain level as directed from LL to –183 inches with systems listed in Table 5 (Band should be –80" to –140")
SRO	Directs SLC initiation when RRC pumps are off (SLC may be started prior to this as RRC-Pumps are off when MT Trips) (REFER TO EVENT 8).
RO	Uses Quick Cards to stops and prevent condensate and feedwater and lines

Event No. 8	Event No. 8				
	up on the startup flow control valves as directed				
		Reports EOP entry on low RPV level			
		Reports Reactor Power as it drops due to lowering level			
		When Reactor Power is LT 5%, marks RPV level to establish an LL			
		Maintains RPV level between LL and –183 inches as directed (–80 inches to –140 inches)			
	SRO	Directs PPM 5.5.10 and 5.5.11 (Tabs B, F) performance for a hydraulic ATWS. (REFER TO EVENT 8)			
COMMEN	TS:				

Event No. 9

Description: Failure of SLC pumps to deliver normal SLC flow to RPV

This event is activated at the beginning of the scenario and realized by the crew when SLC control switches are taken to OPER to initiate SLC

Time	Position	Applicants Actions or Behavior		
T = 50	SRO	Recognizes RRC Pumps have tripped off and directs SLC initiation before SP temp reaches 110°F		
	ATC	Initiates SLC per the quick card:		
		Swaps keys and places two switches to OPER		
		Verifies squib valves fire		
		Verifies RWCU-V-4 closure		
		Verifies flow and SLC tank level		
		Reports reduced SLC flow (about 20 gpm) and initial tank level		

CC	MA	AT.	NTC	

Event No. 10

Description: Insert control rods using PPM 5.5.10 and 5.5.11 Tab B.

This event is initiated by the SRO direction.

Time	Position	Applicants Actions or Behavior		
T = 55	SRO	Directs PPM 5.5.10 and PPM 5.5.11 be performed to insert control rods		
		Performs: PPM 5.5.10 - Override ARI Logic – pulls 2 fuses		
		PPM 5.5.11 Tab B – Starts the second CRD Pump, places the SDV HIGH LEVEL Trip control switch to BYPASS on P603, and determines that scram cannot be reset per PPM 5.5.11		
		PPM 5.5.11 Tab F – The second CRD Pump should already be running, the SDV HIGH LEVEL Trip control switch should already be in BYPASS on P603, and determines that CRD drive header pressure can be established per PPM 5.5.11		
		Directs Instructor to perform back panel operations associated with Tab B and Tab F		

FLOOR OPERATOR: Take direction from BOP to perform back panel steps of Tab B (Step Q-3 to perform attachment 6.1) and Tab F (Step Q-6 to perform attachment 6.2)

When completed with steps, initial by the blocks and inform the BOP operator that they are completed by standing next to Board S and giving the crew an update:

"UPDATE READY - Attachment 6.1 per Tab B and Attachment 6.2 per Tab F of PPM 5.5.11 has been completed, END OF UPDATE."

Event No. 10				
	ВОР	Performs remainder of PPM 5.5.11 Tab B actions to scram/reset/scram:		
		 resets Scram and notes time 		
		• After 2 minutes, checks rod density and inserts a manual reactor scram and informs CRS of results		
		If rods do not insert continues scram/reset/scram Tab B		
	ВОР	Performs remainder actions of PPM 5.5.11 Tab F:		
		 Places RWM Bypass Control Switch to BYPASS 		
		Informs CRS of readiness to drive rods		
	SRO	Directs rod insertion starting at rod 18-19 and inserting every other rod every other row until all rods are inserted skipping the peripheral rods		
	ВОР	Drives Control Rods as directed		
COMMEN	TS: It may ta	ake two S/R/S actions before all rods go in.		

Event	N	Λ	1	1
Lvent	TA	v.	1	1

Description: Control Rods Insert and RPV Level returned to +13" to +54" band

This event is activated when RPV level has been lowered and S/R/S is being performed

Time	Position	Applicants Actions or Behavior
------	----------	--------------------------------

Critical Task is to insert Control Rods by performing PPM 5.5.11 Scram/Reset/Scram

BOOTH OPERATOR: When RPV Level has been lowered and is being controlled in the desired band, and scram/reset/scram is being performed to insert control rods:

Delete/remove malfunctions associated with ATWS

T = 65	ВОР	Checks control rod density	
		Initiates a manual scram	
		Notes Control Rod motion	
		Recognizes and reports All Rods In to the SRO	
	SRO	Directs SLC be stopped	
	RO	Takes control switches out of OPER and observes both SLC pumps stop	
	SRO	Exits PPM 5.1.2 and re-enters PPM 5.1.1	
		Directs RPV level be raised to +13" to +54" band with available systems	
	RO	Raises RPV level into band as directed	

TERMINATION POINT – The scenario will be terminated when RPV level has been returned to normal operating band.

COMMENTS:

TURNOVER INFORMATION

Initial conditions: Columbia is operating at reduced power. Yesterday ASD Channel 1A 1

tripped. The problem has been corrected and Channel 1A1 is ready to be

re-started.

Turnover: Start ASD Channel 1A1 per SOP-RRC-ASD. Step 5.6.1 is complete.

After the channel is started, raise Reactor Power with flow to exit the OPRM Enabled Region. The SNE is working on a reactivity plan to

continue the power increase from that point. Also RCC Pumps need to be swapped for run time equalization. Start RCC-P-1A and place RCC-P-1C

in standby.

SIMULATOR SETUP INSTRUCTIONS

Load IC 220 and load appropriate schedule and Events.

NRC Scenario No. 2

Columbia Generating Station ILC NRC Exam October, 2009

SCHEDULE

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Columbia Generating Station ILC NRC Exam October, 2009

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</ITEM>
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</SCHEDULE>

EVENT

<!-- This file contains a Thunder Simulations Event --> <EVENT>

</EVENT>

Appendix D

FORM ES-D-2

NRC Scenario No. 2 Columbia Generating Station ILC NRC Exam October, 2009



	INS	STRUCTIONAL COVI	ER SHEET
PROGRAM TITLE	OPE	RATIONS TRAINING	
COURSE TITLE	COI	UMBIA GENERATING STATI	ON SIMULATOR EXAMINATION
LESSON TITLE	Surv Surv Cau Offs RHI	e Power with Flow; Start CW-P-eillance Finds an Uncoupled Ro eillance – Controller Failure; Meses Back Pressure To Rise Requite Power; DW Floor Rupture; Re-V-16B Fails Closed; Emergency Well Temp	
		LESSON 1.5 Hours	LIDED
Lesson Plan PQD C		CTIONAL MATERIALS INCL	Rev. No.
Simulator Guide PQ		LO001679	Rev. No. 0
JPM PQD Code			Rev. No.
Exam PQD Code			Rev. No.
DIVISION TITLE	Nuclear Tr	aining	
DEPARTMENT	Operations	Training	
PREPARED BY	Ron Hayde	n	DATE 05/31/09
REVISED BY			DATE
VALIDATED BY			DATE
TECHNICAL REV	IEW _		DATE
INSTRUCTIONAL	REVIEW _		DATE
APPROVED			DATE

Appendix D	NRC Scenario No. 2	FORM ES-D-2	
	Columbia Generating Station ILC NRC Exam October, 2009		
	Operations Training Manager		

Verify materials current IAW SWP-TQS-01 prior to use

NRC Scenario No. 3

Columbia Generating Station ILC NRC Exam October, 2009

Facility: Columbia	NRC Scenario No: 3
Examiners:	Operators:

Initial conditions: Reactor Power is 90%. Power was reduced due to CW-P-1C not being available. The work on CW-P-1C has just been completed.

Turnover: Start CW-P-1C. After the pump start, raise reactor power with Flow to 95% power. The reactivity brief has been performed. At that time stop the power increase and perform OSP-CRD-W701, Control Rod Exercise of Fully Withdrawn Rods (MODE 1) starting with rod 18-59 and working across from left to right and then from top to bottom until all fully withdrawn control rods have been exercised. After CW-P-1C is started perform the Standby Gas Treatment B System Monthly Operability surveillance, OSP-SGT-M702.

Event	Timeline	Event	Event Description
No.		Type*	
1.	T = 0	N (BOP)	Start CW-P-1C
2.	T = 0	R (ATC)	Raise power with Flow
3.	T = 10	R (ATC)	Perform Control Rod Exercise surveillance - OSP-CRD-W701
		C (ATC) TS (SRO)	Second rod is uncoupled and will not re-couple when attempted
4.	T = 10	C (BOP)	Perform SGT B System Operability surveillance – OSP-SGT-M702
		TS (SRO)	Controller Fails when SGT flow is raised
5.	T = 35	M (All)	Minimum Seismic Earthquake
			CW Pipe Rupture outside Protected Area
			MT Back Pressure rise requiring Reactor scram and MT Trip
6.	T = 50	M (All)	Operating Basis Earthquake
			Loss of Offsite Power
7.	T = 55		LOCA
			Drywell Floor Rupture
8.	T = 60	C (SPO)	RHR-P-2A Breaker fails
		C (SRO)	RHR-V-16B Control Power trips and valve still closed
9.	T = 70		Emergency Depressurize due to High Drywell Temperature GT 330°F

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

SCENARIO DESCRIPTION

- EVENT 1 Start CW-P-1C
- EVENT 2 Power will be raised to 95% with Flow
- EVENT 3 Perform OSP-CRD-W701 (Control Rod Exercise Surveillance). Control Rod 22-59 will be uncoupled when checked. When Recoupling is attempted the rod will not recouple. Tech Specs will be referenced and the rod should be driven full in and isolated with cooling flow maintained
- EVENT 4 Perform OSP-SGT-M702 (Standby Gas treatment System B Operability Surveillance). The controller will fail as is when flow reaches 3100 cfm. The unit may be shut down. Tech Specs will be referenced
- EVENT 5 Minimum Seismic Earthquake results in a CW Line rupture outside the protected area. As a result, Main Turbine back pressure will start to rise. The crew should recognize that the event is not recoverable and insert a manual scram and remove the MT from service
- EVENT 6 After the immediate scram actions have been performed, a OBE will occur causing a loss of Startup Power, then a loss of Backup Power (and the Drywell Floor Ruptures and a LOCA starts refer to Event 7)
- EVENT 7 Drywell Floor Rupture and LOCA
- EVENT 8 Inability to Spray (RHR-P-2A Breaker Fails and RHR-V-16B will not open)
- EVENT 9 Emergency Depressurize due to Drywell Temperature when it cannot be restored and maintained LT 330°F
- The scenario will be terminated when RPV level is being returned to normal or as directed by the scenario coordinator

Event No. 1

Description: Start CW-P-1C per SOP-CW-START

This event is initiated by shift turnover

Time	Position	Applicants Actions or Behavior
T = 0	SRO	Directs the start of CW-P-1C per SOP-CW-START
	ВОР	Performs SOP-CW-START Section 5.2:
		• 5.2.1 – Step is N/A
		• 5.2.2 – Contacts OPS4 to verify
		• 5.2.3 – Contacts OPS4 to verify
		Makes plant announcement of CW-P-1C start
		• 5.2.5 – Verifies bay level is GT 441' (is reading 442')
		• 5.2.6 – Places C/S for CW-V-13C and TSW-V-115C to OPEN and when valves start to open releases switch
		• 5.2.7 – Verifies CW-P-1C Blue ready to start light is illuminates
		• 5.2.8 – Verifies 6 towers on line per P&L 4.2
		• 5.2.9 – Step is N/A
		• 5.2.10 – Places CW-P-1C C/S to START and verifies pump starts, discharge valve opens, and CW-V-13C and TSW-V-115C close
		Informs CRS that CW-P-1C is running

COMMENTS:

Event No. 2

Description: Raise Power with Flow

This event is initiated by shift turnover

Time	Position	Applicants Actions or Behavior
T = 0	SRO	Directs ATC to raise power with flow to achieve 95% reactor power at a rate not to exceed 10MWE/min
	ATC	Notes Reactor Power and Main Generator Output Verifies both RRC pumps individual controllers are in AUTO and depresses Master Controller Raise P/B to increase flow/power as directed

COMMENTS:

Event No. 3

Description: Control Rod Exercise. Control Rod 22-59 is not coupled and will not re-couple

This event is initiated by shift turnover and started after Reactor Power is 95%

Time	Position	Applicants Actions or Behavior
T = 10	SRO	Directs performance of OSP-CRD-W701
	ATC	Performs OSP-CRD-W701 section 7.8 for rod 18-59 with coupling check OK: • Selects the rod • Inserts rod 1 notch • Verifies position changes • Continuously withdraws rod • Verifies position change • Verifies coupling integrity (no alarm) • Verifies position 48 • Repeats for next rod
	ATC	Performs for rod 22-59 and receives ROD OVERTRAVEL annunciator (P603.A7.1-8)
	ATC	Acknowledges alarm and refers to ARP: • Notify SM and SNE of condition • Insert control rod 22-59 to position 00 to accomplish recouping • Continuous withdraws 22-59 and receives ROD OVERTRAVEL alarm again

Event No. 3	,	
	SRO	Declares Control Rod 22-59 inoperable
		Refers to Tech Spec 3.1.3 Condition C and verifies total number of slow or inop rods is LE 8 immediately AND Fully insert the inop rod within 3 hours AND disarm the associated CRD within 4 hours
		Directs control rod 22-59 be fully inserted
	SRO	May direct SOP-CRD-HCU section 5.4 be performed for rod 22-59
ROLEPLA	Y: If directed	to isolate rod acknowledge task only - no manipulations need to be done
ROLEPLA	Y: If asked, th	nere are no slow or inop control rods
COMMEN	TS:	

Event No. 4 **Description:** Perform SGT B Monthly Surveillance OSP-SGT-M702 The event is initiated by the turnover and is performed after CW-P-1C is started Time **Position Applicants Actions or Behavior BOOTH OPERATOR**: Have SGT Flow displayed on computer and when flow reaches 3100 CFM ACTIVATE TRIGGER 1 **SRO** Directs Standby Gas Treatment System B monthly operability surveillance, T = 10OSP-SGT-M702 be performed ROLEPLAY – If asked there are no paint fumes, etc. in SGT area and SGT integrity is done Performs OSP-SGT-M702 as directed: **BOP** 7.1.1 - Contacts OPS 2 and verifies no paint fumes 7.1.2 - Contacts OPS 2 and verifies SGT integrity 7.1.3 - Records moisture reading on SGT-MI-4B (reads zero) 7.1.4 - Verifies SGT-V-2B is open (Rx Bldg inlet) 7.1.5 - Verifies SGT-V-3B1 is open (fan 1B2 inlet) 7.1.6 - Depresses BISI Manual Out of Service pushbutton 7.1.7 - Informs CRS to enters SGT system B as inoperable in the TS surveillance log 7.1.8 - Places SGT-DPIC-1B2 in MANUAL 7.1.9 - Adjusts SGT-DPIC-1B2 output to minimum (100%) 7.1.10 - Places SGT-EHC-1B2 control switch to ON 7.1.11 - Verifies SGT-FN-1B2 auto starts 10 sec. after heaters energize 7.1.12 - Promptly opens SGT-V-5B2 (exhaust to stack) 7.1.13 - Slowly adjusts SGT-DPIC-1B2 to obtain 4800 CFM \pm 480CFM

BOOTH OPERATOR: When flow reaches 3100 CFM ACTIVATE TRIGGER 1

Event No. 4	4	
	ВОР	Observes and reports to the CRS that the DPIC cannot be adjusted and appears to have failed-as-is with SGT flow at about 3100 CFM
	SRO	May directs BOP to shutdown the SGT system and restore the lineup
		Refers to Tech Specs due to DPIC failure and determines TS 3.6.4.3 Condition A applies – Restore SGT B within 7 Days
		May contact Production/Work Control/SSS
	: If asked to s	secure or leave SGT-B running ask the crew to leave it running so it out.
	BOP	May use SOP-SGT-SHUTDOWN to secure SGT or use surveillance and do steps in reverse order
		Per SOP-SGT-SHUTDOWN Section 5.1.2:
		Take the following switches to PULL TO LOCK: SGT-EHC-1B2 SGT-V-2B SGT-V-3B1 SGT-V-2B2 SGT-V-5B1 SGT-V-5B2
		When SGT-EHC-1B1 and SGT-FN-1B1 starts then place SGT-EHC-1B1 to OFF
COMMEN	TS:	

NRC Scenario No. 3

Columbia Generating Station ILC NRC Exam October, 2009

Event No. 5

Description: Minimum Seismic Earthquake results in Circ Water Rupture Outside Protected Area requiring a Reactor Scram and Main Turbine Trip

This event is initiated when the two surveillances are completed and Tech Specs have been reviewed by **ACTIVATING TRGGER 2 – BOOTH OPERATOR SEE BELOW INSTRUCTIONS**

Time	Position	Applicants Actions or Behavior
		Start EQ track on lowest volume and run for 5 seconds and GER 2. Allow EQ track to run another 5 seconds then stop it
T = 35	BOP	Acknowledges Minimum Seismic Alarm, pulls ARP and refers CRS to ABN-EARTHQUAKE
		Investigates and reports indications on Bd. L (16 Amber and no red lights)
	Y: 30 seconds at e just a minute a	fter EQ contact Control Room as OPS 4 and report that you felt the
	SRO	Directs announcement per ABN-EARTHQUAKE be performed
	ВОР	Makes announcement and directs SAS (meets booth operator at booth door) to repeat on the Maintenance and Security radio channels
and report	a large piping b	nouncement has been performed contact the control room as OPS 4 reak between the cooling towers has resulted in a large volume of water at into the desert
	SRO	May direct announcement concerning pipe rupture be performed
	ВОР	Makes announcement if directed
		Reports lowering MWe output and rising MT Back Pressure
		an attempt to keep plant/Main Turbine on line longer
	SRO	Determines that it is necessary to trip the Main Turbine due to rising back pressure
		Conducts a brief on scram and MT trip

Event No. 5	ī	
	SRO	May direct a RRC flow reduction prior to directing the Reactor Scram Directs ATC to insert a manual scram
	ATC	Lowers Core Flow if directed Announces "Listen up for the scram report"
		Performs immediate scram actions:
		 Places Reactor Mode Switch in SHUTDOWN Monitors Power, Pressure, and Level Verifies all rods inserted Inserts IRMs and SRMs by depressing INSERT P/B Reports EOP entry on low RPV water level
	SRO	May direct MT Trip prior to its auto trip Enters PPM 5.1.1 on Low RPV Level and directs restoration of RPV level to +13 inches to +54 inches (a -40" to +50 " band should be given) with RCIC and/or Condensate and Feed system
COMMEN	TS:	

Event No. 6

Description: Operating Basis Earthquake; Loss of Startup Power (10 sec. TD); Loss of Backup Power (120 sec. TD)

This event is initiated by **ACTIVATING TRIGGER 3–BOOTH OPERATOR SEE BELOW**

INSTRUCTIONS			
Time	Position	Applicants Actions or Behavior	
to full vol	BOOTH OPERATOR: Start EQ track on lowest volume and take 19 seconds to get to full volume. After 4 seconds <u>ACTIVATE TRIGGER 3</u> . Allow EQ track to run another 5 seconds after reaching max volume and then stop it		
Critical ste	ep is to initiate sy	vstems required to restore RPV level back to +13" to =54"	
T = 55	ВОР	Reports OBE annunciator, pulls ARP and refers SRO to ABN-EARTHQUAKE.	
		Investigates and reports indications on Bd. L (all red and yellow lights illuminated)	
	ВОР	Investigates electrical boards when the Startup Transformer locks out and updates crew that Startup power is locked out and Backup power is on SM-7 and SM-8. SM-4 is powered from DG-3	
	SRO	Due to the loss of feedwater directs RPV level restoration with RCIC and/or HPCS and/or CRD	
	ATC	Initiates RCIC and or HPCS using quick cards	
		Restarts CRD pump	
		Uses RCIC and/or HPCS to maintain RPV Level	
	ВОР	Investigates electrical board when Backup Transformer locks out and updates crew that Backup power is locked out and SM-7 and SM-8 are powered from Diesel Generators	
	SRO	Directs RPV Pressure be maintained with SRVs with a band of 800 to	

1000 psig

	BOP/ATC	Cycles SRVs as necessary to maintain RPV Pressure between 800 and 1000 psig (When LOCA starts SRVs will not have to be cycled to control RPV pressure)
	SRO	Directs actuations for low RPV level be verified
	ВОР	Verifies and reports actuations complete
	BOP/ATC	Recognizes that CAS compressors and TSW are not running.
		Directs OPS 3 to place FW on CAS Heat Exchanger cooling and reset and restart CAS compressors A and B
		nen directed and after appropriate time delay, place firewater on CAS AS compressors and report results to control room
COMMEN	TS:	

Event No. 7

Description: LOCA (on a 120 sec. TD); Drywell Floor Failure; RHR-P-2A Breaker Fails to Close

This event is initiated from trigger 3 which has already been activated

Time	Position	Applicants Actions or Behavior
T = 55	ВОР	Reports Drywell Pressure rising and at 1.68 psig reports EOP entry into PPM 5.2.1 (reports others as they occur – SP/T, SP/L, DW/T)
	SRO	Directs actuations for 1.68 psig DW/P be verified
	ВОР	Verifies actuations for 1.68 psig and notes RHR-P-2A is not running
		Attempts to manually close RHR-P-2A breaker and notes it does not close
		Reports 1.68 actuation findings to CRS
	ВОР	Observes that Wetwell and Drywell pressures are approximately the same and informs the CRS
		Reports Wetwell pressure when GT 2 psig
	SRO	Directs Wetwell Sprays be initiated with RHR-P-2B
		May direct SP Cooling be initiated
		Directs sprays be terminated when WW/P drops below 1.68 psig
	ВОР	Initiates Wetwell sprays using quick card and opens RHR-V-27B
		Initiates Suppression Pool Cooling using quick card if directed
	Reports Wetwell sprays initiated	

COMMENTS:

Event No. 8

Description: Inability to Spray Drywell

This event is initiated from trigger 3 which has already been activated

Time	Position	Applicants Actions or Behavior
T = 65	ВОР	Reports Drywell Temperature as it approaches 285°F (It takes 6 minutes to reach 285°F after trigger 3 is initiated)
	SRO	Ensures parameters are within DSIL, WW/L LT 51', RRC pumps are stopped and directs Drywell Cooling Fans be secured
	ATC/BOP	Stops Drywell Cooling Fans and reports completion to CRS
	SRO	Directs Drywell sprays be initiated with RHR-P-2B
	ВОР	Using quick card, verifies within DSIL and opens RHR-V-17B and attempts to open RHR-V-16B
		Notes and reports that RHR-V-16B will not open with C/S was turned to start
	SRO	May contact Production/Work Control/OPS1/OPS2 to manually open RHR-V-16B
	Y: If asked to r it will not open	manually open RHR-V-16B wait 10 minutes and inform the Control
	SRO	Sets Drywell Temperature as a KEY parameters due to inability to spray drywell
	ATC/BOP	Reports Drywell temperature as it rises towards 330°F
		Reports trend of parameters as it approaches PSP limits
COMMEN	 TS•	

Event No. 9

Description: EMERGENCY DEPRESSURIZATION when Drywell Temperature reaches 330°F (It takes 10 minutes to get to 330°F from trigger 3 initiation)

This event is initiated when it is determined that Drywell Temperature cannot be restored and maintained LT $330^{\circ}F$

Time	Position	Applicants Actions or Behavior		
Critical step is to initiate an EMERGENCY DEPRESSURIZATION when drywell Temperature cannot be restored and maintained LT 330°F				
T = 60	ВОР	Reports Drywell Temperature as it approaches 330°F		
	SRO	When Drywell Temperature cannot be restored and maintained below 330°F, determines that an Emergency Depressurization is required:		
		Takes PPM 5.1.1override to PPM 5.1.3, Emergency RPV Depressurization		
	ATC/BOP	Stops ECCS injection as necessary to maintain RPV level		
		Directs ECCS pumps not required for adequate core cooling be stopped from injecting		
	SRO	Requests Wetwell Level and when reported GT 17 foot, directs seven SRVs, ADS preferred, be opened		
	ATC/BOP	Opens seven ADS SRVs as directed and reports completion to CRS		

Appendix D

NRC Scenario No. 3

FORM ES-D-2

Columbia Generating Station ILC NRC Exam October, 2009

	SRO	Directs RPV/L maintenance +13 inches to +54 inches.				
Termination Cue: The scenario can be terminated when the reactor has been Emergency Depressurized and RPV level is under control in the band of +13 inches to +54 inches or as directed by the scenario coordinator						
COMMENTS:						

NRC Scenario No. 3

Columbia Generating Station ILC NRC Exam October, 2009

TURNOVER INFORMATION

Initial conditions: Reactor Power is 90%. Power was reduced due to CW-P-1C not being available. The work on CW-P-1C has just been completed.

Turnover:

Start CW-P-1C. After the pump start, raise reactor power with Flow to 95% power. The reactivity brief has been performed. At that time stop the power increase and perform OSP-CRD-W701, Control Rod Exercise of Fully Withdrawn Rods (MODE 1) starting with rod 18-59 and working across from left to right and then from top to bottom until all fully withdrawn control rods have been exercised. After CW-P-1C is started perform the Standby Gas Treatment B System Monthly Operability surveillance, OSP-SGT-M702.

SIMULATOR SETUP INSTRUCTIONS

Reset to IC 219

Set up earthquake machine on lowest volume and paused
Ensure all events and schedules windows are closed
Place Simulator in Run
Open Schedule window and open LO001679
Open Director file

NRC Scenario No. 3

Columbia Generating Station ILC NRC Exam October, 2009

SCHEDULE

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<!-- This file contains a Thunder Simulations Schedule -->
<SCHEDULE>
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            <TIME>0</TIME>
            <ACTION>Insert malfunction MAL-RMC006-2259</ACTION>
            <DESCRIPTION>ROD 2259 UNCOUPLED</DESCRIPTION>
      </TTEM>
      <ITEM row = 2>
            <TIME>0</TIME>
            <ACTION>Insert malfunction BKR-RHR001 to FA_AS_IS</ACTION>
            <DESCRIPTION>RHR-P-2A Breaker Fails As Is/DESCRIPTION>
      </ITEM>
      <ITEM row = 3>
            <TIME>0</TIME>
            <ACTION>Insert malfunction MOV-RHR009F to FAIL AS IS</ACTION>
            <DESCRIPTION>RHR-V-16B Fails closed/DESCRIPTION>
      </ITEM>
      <ITEM row = 5>
            <TIME>0</TIME>
            <EVENT>1</EVENT>
            <ACTION>Insert malfunction CNH-SCN007A on event 1</ACTION>
            <DESCRIPTION>SGT-DPIC-1B2 FAN 1B-2 FLOW CONTROLLER FAIL AS IS/DESCRIPTION>
      </ITEM>
      <ITEM row = 7>
            <TIME>0</TIME>
            <EVENT>2</EVENT>
            <ACTION>Insert malfunction MAL-RWB001 to 0.1 on event 2</ACTION>
            <DESCRIPTION>Minimum Seismic EQ</DESCRIPTION>
      </ITEM>
      <ITEM row = 8>
            <TIME>0</TIME>
            <EVENT>2</EVENT>
            <ACTION>Insert malfunction MAL-CFW003 to 2400 in 600 on event 2</ACTION>
            <DESCRIPTION>CONDENSER AIR LEAK</DESCRIPTION>
      </ITEM>
      <ITEM row = 10>
            <TIME>0</TIME>
            <EVENT>3</EVENT>
            <ACTION>Insert malfunction MAL-RWB001 to 0.2 on event 3</ACTION>
            <DESCRIPTION>EARTHQUAKE</DESCRIPTION>
      </ITEM>
      <ITEM row = 11>
            <TIME>0</TIME>
            <EVENT>3</EVENT>
            <ACTION>Insert malfunction MAL-OED001 after 10 on event 3</ACTION>
            <DESCRIPTION>LOCKOUT TR-S 10 seconds after OBE</DESCRIPTION>
      </ITEM>
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Columbia Generating Station ILC NRC Exam October, 2009

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<ITEM row = 12>
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           <EVENT>3</EVENT>
           <ACTION>Insert malfunction MAL-OED003 after 120 on event 3</ACTION>
           <DESCRIPTION>LOCKOUT TR-B 120 seconds after OBE</DESCRIPTION>
     </ITEM>
     <ITEM row = 13>
           <TIME>0</TIME>
           <EVENT>3</EVENT>
           <ACTION>Insert malfunction MAL-PCN006 on event 3</ACTION>
           <DESCRIPTION>Drywell Floor Failure
     </ITEM>
     <ITEM row = 14>
           <TIME>0</TIME>
           <EVENT>3</EVENT>
           <ACTION>Insert malfunction MAL-RRS009A after 120 to 500000 in 1200 on event
3</ACTION>
           <DESCRIPTION>Steam Line break in Drywell 2 minutes after OBE</DESCRIPTION>
     </ITEM>
</SCHEDULE>
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