

Facility:		ILT Class 08-01 NRC										Date of Exam:					
Tier	Group	RO K/A Category Points											SRO-Only Points				
		K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	Total	A2	G*	Total	
1. Emergency & Plant Evolutions	1	5	2	3				4	3			3	20	4	3	7	
	2	2	1	1				1	1			1	7	1	2	3	
	Tier Totals	7	3	4				5	4			4	27	5	5	10	
2. Plant Systems	1	2	2	2	3	2	3	2	2	2	3	3	26	3	2	5	
	2	2	1	1	2	1	1	0	1	1	1	1	12	0	1	3	
	Tier Totals	4	3	3	5	3	4	2	3	3	4	4	38	4	4	8	
3. Generic Knowledge & Abilities Categories				1		2		3		4		10	1	2	3	4	7
				2		3		2		3			2	2	2	1	
<p>Note: 1. Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two).</p> <p>2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ± 1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO-only exam must total 25 points.</p> <p>3. Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted and justified; operationally important, site-specific systems that are not included on the outline should be added. Refer to section D.1.b of ES-401, for guidance regarding elimination of inappropriate K/A statements.</p> <p>4. Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution.</p> <p>5. Absent a plant specific priority, only those KAs having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively.</p> <p>6. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.</p> <p>7.* The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system. Refer to Section D.1.b of ES-401 for the applicable K/A's</p> <p>8. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IR) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above. If fuel handling equipment is sampled in other than Category A2 or G* on the SRO-only exam, enter it on the left side of Column A2 for Tier 2, Group 2 (Note #1 does not apply). Use duplicate pages for RO and SRO-only exams.</p> <p>9. For Tier 3, select topics from Section 2 of the K/A Catalog, and enter the K/A numbers, descriptions, IRs, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10CFR55.43</p>																	

ILT Class 08-01 NRC
Written Examination Outline
Emergency and Abnormal Plant Evolutions – Tier 1 Group 1

EAPE # / Name Safety Function	K1	K2	K3	A1	A2	G	K/A Topic(s)	Imp.	Q#
295003 Partial or Complete Loss of AC / 6					X		AA2.03 - Ability to determine and/or interpret the following as they apply to PARTIAL OR COMPLETE LOSS OF A.C. POWER : Battery status: Plant-Specific	3.5	76
600000 Plant Fire On-site / 8					X		AA2.05 - Ability to determine and interpret the following as they apply to PLANT FIRE ON SITE: Ventilation alignment necessary to secure affected area	3.0	77
295023 Refueling Acc Cooling Mode / 8					X		AA2.04 - Ability to determine and/or interpret the following as they apply to REFUELING ACCIDENTS : Occurrence of fuel handling accident	4.1	78
295023 Refueling Acc Cooling Mode / 8						X	2.2.37 - Equipment Control: Ability to determine operability and / or availability of safety related equipment.	4.6	79
295021 Loss of Shutdown Cooling / 4						X	2.2.37 - Equipment Control: Ability to determine operability and / or availability of safety related equipment.	4.6	80
295004 Partial or Total Loss of DC Pwr / 6						X	2.2.44 - Equipment Control: Ability to interpret control room indications to verify the status and operation of a system, and understand how operator actions and directives effect plant and system conditions.	4.4	81
295006 SCRAM / 1					X		AA2.01 - Ability to determine and/or interpret the following as they apply to SCRAM : Reactor power	4.6	82
295005 Main Turbine Generator Trip / 3	X						AK1.03 - Knowledge of the operational implications of the following concepts as they apply to MAIN TURBINE GENERATOR TRIP : Pressure effects on reactor level	3.5	39
295037 SCRAM Conditions Present and Reactor Power Above APRM Downscale or Unknown / 1	X						EK1.01 - Knowledge of the operational implications of the following concepts as they apply to SCRAM CONDITION PRESENT AND REACTOR POWER ABOVE APRM DOWNSCALE OR UNKNOWN: Reactor pressure effects on reactor power	4.1	40
295026 Suppression Pool High Water Temp. / 5	X						EK1.01 - Knowledge of the operational implications of the following concepts as they apply to SUPPRESSION POOL HIGH WATER TEMPERATURE : Pump NPSH	3.0	41
295018 Partial or Total Loss of CCW / 8		X					AK2.02 - Knowledge of the interrelations between PARTIAL OR COMPLETE LOSS OF COMPONENT COOLING WATER and the following: Plant operations	3.4	42
295005 Main Turbine Generator Trip / 3	X						AK1.03 - Knowledge of the operational implications of the following concepts as they apply to MAIN TURBINE GENERATOR TRIP : Pressure effects on reactor level	3.5	43
295031 Reactor Low Water Level / 2		X					EK2.02 - Knowledge of the interrelations between REACTOR LOW WATER LEVEL and the following: Reactor pressure	3.8	44
295016 Control Room Abandonment / 7			X				AK3.01 - Knowledge of the reasons for the following responses as they apply to CONTROL ROOM ABANDONMENT : Reactor SCRAM	4.1	45

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EAPE # / Name Safety Function	K1	K2	K3	A1	A2	G	K/A Topic(s)	Imp.	Q#
295005 Main Turbine Generator Trip / 3			X				AK3.02 - Knowledge of the reasons for the following responses as they apply to MAIN TURBINE GENERATOR TRIP: Recirculation pump downshift/trip: Plant-Specific	3.4	46
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4			X				AK3.03 - Knowledge of the reasons for the following responses as they apply to PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION : Idle loop flow	2.8	47
700000 Generator Voltage and Electric Grid Disturbances				X			AA1.05 - Ability to operate and/or monitor the following as they apply to GENERATOR VOLTAGE AND ELECTRIC GRID DISTURBANCES: Engineered safety features.	3.9	48
295006 SCRAM / 1				X			AA1.01 - Ability to operate and/or monitor the following as they apply to SCRAM : RPS	4.2	49
295003 Partial or Complete Loss of AC / 6				X			AA1.04 - Ability to operate and/or monitor the following as they apply to PARTIAL OR COMPLETE LOSS OF A.C. POWER : D.C. electrical distribution system	3.6	50
295028 High Drywell Temperature / 5					X		EA2.05 - Ability to determine and/or interpret the following as they apply to HIGH DRYWELL TEMPERATURE : Torus/suppression chamber pressure: Plant-Specific	3.6	51
295004 Partial or Total Loss of DC Pwr / 6					X		AA2.03 - Ability to determine and/or interpret the following as they apply to PARTIAL OR COMPLETE LOSS OF D.C. POWER : Battery voltage	2.8	52
295038 High Off-site Release Rate / 9					X		EA2.03 - Ability to determine and/or interpret the following as they apply to HIGH OFF-SITE RELEASE RATE : Radiation levels	3.5	53
295030 Low Suppression Pool Water Level / 5						X	2.4.49 - Emergency Procedures / Plan: Ability to perform without reference to procedures those actions that require immediate operation of system components and controls.	4.6	54
295025 High Reactor Pressure / 3						X	2.1.32 - Conduct of Operations: Ability to explain and apply all system limits and precautions.	3.8	55
295021 Loss of Shutdown Cooling / 4						X	2.2.25 - Equipment Control: Knowledge of bases in technical specifications for limiting conditions for operations and safety limits.	3.2	56
295023 Refueling Accident / 8	X						AK1.02 - Knowledge of the operational implications of the following concepts as they apply to REFUELING ACCIDENTS : Shutdown margin	3.2	57
700000 Generator Voltage and Electric Grid Disturbances				X			AA1.03 - Ability to operate and/or monitor the following as they apply to GENERATOR VOLTAGE AND ELECTRIC GRID DISTURBANCES: Voltage regulator controls.	3.8	58
K/A Category Totals:	5	2	3	4	3/4	3/3	Group Point Total:		20/7

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Written Examination Outline
Emergency and Abnormal Plant Evolutions – Tier 1 Group 2

EAPE # / Name Safety Function	K1	K2	K3	A1	A2	G	K/A Topic(s)	Imp.	Q#
295008 High Reactor Water Level / 2					X		AA2.04 - Ability to determine and/or interpret the following as they apply to HIGH REACTOR WATER LEVEL : Heatup rate: Plant-Specific	3.3	83
295007 High Reactor Pressure / 3						X	2.2.40 - Equipment Control: Ability to apply technical specifications for a system.	4.7	84
295022 Loss of CRD Pumps / 1						X	2.1.32 - Conduct of Operations: Ability to explain and apply all system limits and precautions.	4.0	85
295035 Secondary Containment High Differential Pressure / 5	X						EK1.02 - Knowledge of the operational implications of the following concepts as they apply to SECONDARY CONTAINMENT HIGH DIFFERENTIAL PRESSURE : Radiation release	3.7	59
295033 High Secondary Containment Area Radiation Levels / 9		X					EK2.02 - Knowledge of the operational implications of the following concepts as they apply to HIGH SECONDARY CONTAINMENT AREA RADIATION LEVELS : Process radiation monitoring system	3.8	60
295015 Incomplete SCRAM / 1			X				AK3.01 - Knowledge of the reasons for the following responses as they apply to INCOMPLETE SCRAM : Bypassing rod insertion blocks	3.4	61
295013 High Suppression Pool Temperature / 5				X			AA1.01 - Ability to operate and/or monitor the following as they apply to HIGH SUPPRESSION POOL TEMPERATURE : Suppression pool cooling	3.9	62
295012 High Drywell Temperature / 5					X		AA2.01 - Ability to determine and/or interpret the following as they apply to HIGH DRYWELL TEMPERATURE : Drywell temperature	3.8	63
295008 High Reactor Water Level / 2						X	2.4.20 - Emergency Procedures / Plan: Knowledge of operational implications of EOP warnings, cautions, and notes.	3.8	64
295032 High Secondary Containment Area Temperature / 5	X						EK1.02 - Knowledge of the operational implications of the following concepts as they apply to HIGH SECONDARY CONTAINMENT AREA TEMPERATURE: Radiation releases	3.6	65
K/A Category Totals:	2	1	1	1	1/1	1/2	Group Point Total:		7/3

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Written Examination Outline
Plant Systems – Tier 2 Group 1

System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A2	A 3	A 4	G		Imp .	Q#
223002 PCIS/Nuclear Steam Supply Shutoff								X				A2.08 - Ability to (a) predict the impacts of the following on the PRIMARY CONTAINMENT ISOLATION SYSTEM/NUCLEAR STEAM SUPPLY SHUT-OFF ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Surveillance testing	3.1	86
264000 EDGs								X				A2.01 - Ability to (a) predict the impacts of the following on the EMERGENCY GENERATORS (DIESEL/JET) ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Parallel operation of emergency generator	3.6	87
217000 RCIC											X	2.4.21 - Emergency Procedures / Plan: Knowledge of the parameters and logic used to assess the status of safety functions, such as reactivity control, core cooling and heat removal, reactor coolant system integrity, containment conditions, radioactivity release control, etc.	4.6	88
264000 EDGs											X	2.4.18 - Emergency Procedures / Plan: Knowledge of the specific bases for EOPs.	4.0	89
206000 HPCI								X				A2.11 - Ability to (a) predict the impacts of the following on the HIGH PRESSURE COOLANT INJECTION SYSTEM ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Low reactor water level: BWR-2,3,4	4.2	90
215005 APRM / LPRM	X											K1.01 - Knowledge of the physical connections and/or cause- effect relationships between AVERAGE POWER RANGE MONITOR/LOCAL POWER RANGE MONITOR SYSTEM and the following: RPS	4.0	1
263000 DC Electrical Distribution	X											K1.03 - Knowledge of the physical connections and/or cause- effect relationships between D.C. ELECTRICAL DISTRIBUTION and the following: Battery ventilation	2.6	2
217000 RCIC		X										K2.02 - Knowledge of electrical power supplies to the following: RCIC initiation signals (logic)	2.8	3

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Plant Systems – Tier 2 Group 1

System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A2	A 3	A 4	G		Imp .	Q#
400000 Component Cooling Water		X										K2.02 - Knowledge of electrical power supplies to the following: CCW valves	2.9	4
212000 RPS			X									K3.08 - Knowledge of the effect that a loss or malfunction of the REACTOR PROTECTION SYSTEM will have on following: Reactor coolant primary system integrity	3.6	5
205000 Shutdown Cooling			X									K3.05 - Knowledge of the effect that a loss or malfunction of the SHUTDOWN COOLING SYSTEM (RHR SHUTDOWN COOLING MODE) will have on following: Fuel pool cooling assist: Plant-Specific	2.6	6
209001 LPCS				X								K4.10 - Knowledge of LOW PRESSURE CORE SPRAY SYSTEM design feature(s) and/or interlocks which provide for the following: Testability of all operable components	2.8	7
239002 SRVs				X								K4.05 - Knowledge of RELIEF/SAFETY VALVES design feature(s) and/or interlocks which provide for the following: Allows for SRV operation from more than one location: Plant-Specific	3.6	8
206000 HPCI					X							K5.05 - Knowledge of the operational implications of the following concepts as they apply to HIGH PRESSURE COOLANT INJECTION SYSTEM : Turbine speed control: BWR-2,3,4	3.3	9
215003 IRM					X							K5.01 - Knowledge of the operational implications of the following concepts as they apply to INTERMEDIATE RANGE MONITOR (IRM) SYSTEM : Detector operation	2.6	10
300000 Instrument Air						X						K6.07 - Knowledge of the effect that a loss or malfunction of the following will have on the INSTRUMENT AIR SYSTEM: Valves	2.5	11
223002 PCIS/Nuclear Steam Supply Shutoff						X						K6.08 - Knowledge of the effect that a loss or malfunction of the following will have on the PRIMARY CONTAINMENT ISOLATION SYSTEM/NUCLEAR STEAM SUPPLY SHUT-OFF : Reactor protection system	3.5	12

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Written Examination Outline
Plant Systems – Tier 2 Group 1

System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A2	A 3	A 4	G		Imp	Q#
264000 EDGs							X					A1.09 - Ability to predict and/or monitor changes in parameters associated with operating the EMERGENCY GENERATORS (DIESEL/JET) controls including: Maintaining minimum load on emergency generator (to prevent reverse power)	3.0	13
262001 AC Electrical Distribution							X					A1.04 - Ability to predict and/or monitor changes in parameters associated with operating the A.C. ELECTRICAL DISTRIBUTION controls including: Load currents	2.7	14
218000 ADS								X				A2.03 - Ability to (a) predict the impacts of the following on the AUTOMATIC DEPRESSURIZATION SYSTEM ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Loss of air supply to ADS valves: Plant-Specific	3.4	15
400000 Component Cooling Water								X				A2.03 - Ability to (a) predict the impacts of the following on the CCWS and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal operation: High/low CCW temperature	2.9	16
259002 Reactor Water Level Control									X			A3.04 - Ability to monitor automatic operations of the REACTOR WATER LEVEL CONTROL SYSTEM including: Changes in reactor feedwater flow	3.2	17
262002 UPS (AC/DC)									X			A3.01 - Ability to monitor automatic operations of the UNINTERRUPTABLE POWER SUPPLY (A.C./D.C.) including: Transfer from preferred to alternate source	2.8	18
211000 SLC										X		A4.03 - Ability to manually operate and/or monitor in the control room: Explosive valves firing circuit status	4.1	19
215004 Source Range Monitor										X		A4.07 - Ability to manually operate and/or monitor in the control room: Verification of proper functioning/ operability	3.4	20
206000 HPCI											X	2.4.2 - Emergency Procedures / Plan: Knowledge of system set points, interlocks and automatic actions associated with EOP entry conditions.	4.5	21

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Plant Systems – Tier 2 Group 1

System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A2	A 3	A 4	G		Imp .	Q#
261000 SGTS											X	2.4.8 - Emergency Procedures / Plan: Knowledge of how abnormal operating procedures are used in conjunction with EOP's.	3.8	22
262001 AC Electrical Distribution						X						K6.02 - Knowledge of the effect that a loss or malfunction of the following will have on the A.C. ELECTRICAL DISTRIBUTION: Off-site power	3.6	23
218000 ADS											X	2.1.23 - Conduct of Operations: Ability to perform specific system and integrated plant procedures during all modes of plant operation.	4.3	24
215003 IRM										X		A4.07 - Ability to manually operate and/or monitor in the control room: Verification of proper functioning/ operability	3.6	25
212000 RPS				X								K4.11 - Knowledge of REACTOR PROTECTION SYSTEM design feature(s) and/or interlocks which provide for the following: Operation with shorting links removed: Plant-Specific	3.3	26
K/A Category Totals:	2	2	2	3	2	3	2	2/3	2	3	3/2	Group Point Total:	26/5	

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Written Examination Outline
Plant Systems – Tier 2 Group 2

System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A2	A 3	A 4	G		Imp.	Q #
239003 MSIV Leakage Control								X				A2.10 - Ability to (a) predict the impacts of the following on the MSIV LEAKAGE CONTROL SYSTEM ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: A.C. distribution power failures: BWR-4,5,6	3.0	91
290002 Reactor Vessel Internals											X	2.2.42 - Equipment Control:: Ability to recognize system parameters that are entry-level conditions for Technical Specifications.	4.6	92
268000 Radwaste											X	2.2.44 - Equipment Control: Ability to interpret control room indications to verify the status and operation of a system, and understand how operator actions and directives effect plant and system conditions.	4.4	93
202002 Recirculation Flow Control	X											K1.01 - Knowledge of the physical connections and/or cause- effect relationships between RECIRCULATION FLOW CONTROL SYSTEM and the following: Recirculation system	3.5	27
230000 RHR/LPCI: Torus/Pool Spray Mode		X										K2.02 - Knowledge of electrical power supplies to the following: Pumps	2.8	28
201001 CRD Hydraulic			X									K3.03 - Knowledge of the effect that a loss or malfunction of the CONTROL ROD DRIVE HYDRAULIC SYSTEM will have on following: Control rod drive mechanisms	3.1	29
241000 Reactor/Turbine Pressure Regulator				X								K4.19 - Knowledge of REACTOR/TURBINE PRESSURE REGULATING SYSTEM design feature(s) and/or interlocks which provide for the following: Steam bypass valve control	3.6	30
216000 Nuclear Boiler Inst.					X							K5.12 - Knowledge of the operational implications of the following concepts as they apply to NUCLEAR BOILER INSTRUMENTATION : Effects on level indication due to rapid changes in void fraction	3.2	31
223001 Primary CTMT and Aux.						X						K6.14 - Knowledge of the effect that a loss or malfunction of the following will have on the PRIMARY CONTAINMENT SYSTEM AND AUXILIARIES : RHR/LPCI	3.6	32

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Plant Systems – Tier 2 Group 2

System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A2	A 3	A 4	G		Imp.	Q #
214000 RPIS	X											K1.01 - Knowledge of the physical connections and/or cause- effect relationships between ROD POSITION INFORMATION SYSTEM and the following: RWM: Plant-Specific	3.0	33
268000 Radwaste								X				A2.01 - Ability to (a) predict the impacts of the following on the RADWASTE ; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: System rupture	2.9	34
215002 RBM									X			A3.02 - Ability to monitor automatic operations of the ROD BLOCK MONITOR SYSTEM including: Meters and recorders: BWR-3,4,5	3.1	35
290001 Secondary CTMT										X		A4.09 - Ability to manually operate and/or monitor in the control room: System status lights and alarms: Plant-Specific	3.2	36
290003 Control Room HVAC										X		2.1.31 - Conduct of Operations: Ability to locate control room switches, controls, and indications, and to determine that they correctly reflect the desired plant lineup.	4.6	37
219000 RHR/LPCI: Torus/Pool Cooling Mode				X								K4.09 - Knowledge of RHR/LPCI: TORUS/SUPPRESSION POOL COOLING MODE design feature(s) and/or interlocks which provide for the following: Heat exchanger cooling	3.3	38
K/A Category Totals:	2	1	1	2	1	1	0	1/1	1	1	1/2	Group Point Total:	12/3	

Facility:		ILT Class 08-01 NRC		Date:			
Category	K/A #	Topic	RO		SRO-Only		
			IR	Q#	IR	Q#	
1. Conduct of Operations	2.1.20	Ability to interpret and execute procedure steps.			4.6	94	
	2.1.41	Knowledge of the refueling process.			3.7	98	
	2.1.2	Knowledge of operator responsibilities during all modes of plant operation.	4.1	66			
	2.1.36	Knowledge of procedures and limitations involved in core alterations.	3.0	67			
	Subtotal			2		2	
2. Equipment Control	2.2.37	Ability to determine operability and / or availability of safety related equipment.			4.6	95	
	2.2.23	Ability to track Technical Specification limiting conditions for operations.			4.6	99	
	2.2.6	Knowledge of the process for making changes to procedures.	3.0	68			
	2.2.17	Knowledge of the process for managing maintenance activities during power operations, such as risk assessments, work prioritizaion, coordination with the transmission system operator.	2.6	69			
	2.2.41	Ability to obtain and interpret station electrical and mechanical drawings.	3.5	74			
	Subtotal			3		2	
3. Radiation Control	2.3.15	Knowledge of radiation monitoring systems, such as fixed radiation monitors and alarms, portable survey instruments, personnel monitoring equipment, etc.			3.1	96	
	2.3.4	Knowledge of radiation exposure limits under normal or emergency conditions.			3.7	100	
	2.3.7	Ability to comply with radiation work permit requirements during normal or abnormal conditions.	3.5	70			
	2.3.12	Knowledge of Radialogical Safety Principles pertaining to licensed operator duties, such as containment entry requirements, fuel handling responsibilities, access to locked high-radiation areas, aligning filters, etc.	3.2	71			

	Subtotal			2		2
4. Emergency Procedures / Plan	2.4.23	Knowledge of the bases for prioritizing emergency procedure implementation during emergency operations.			4.4	97
	2.4.22	Knowledge of the bases for prioritizing safety functions during abnormal/emergency operations.	3.6	72		
	2.4.46	Ability to verify that the alarms are consistent with the plant conditions.	4.2	73		
	2.4.35	Knowledge of local auxiliary operator tasks during emergency and the resultant operational effects.	3.8	75		
	Subtotal			3		1
Tier 3 Point Total				10		7

Tier / Group	Randomly Selected K/A	Reason for Rejection
1 / 1	295027 / EK1.01	Cooper Station has Mark 1 containment.
2 / 1	207000 / K2.02	Cooper Nuclear Station is a BWR 4 and does not have Isolation Condensers, this K/A specific to BWR 2, 3.
2 / 1	207000 / K6.01	Cooper Nuclear Station is a BWR 4 and does not have Isolation Condensers, this K/A specific to BWR 2, 3.
2 / 1	207000 / K4.04	Cooper Nuclear Station is a BWR 4 and does not have Isolation Condensers, this K/A specific to BWR 2, 3.
2 / 2	286000 / K6.05	Cooper Nuclear Station has specific fire pumps that draw water from a storage tank. This water is supplied via a well system and does not come to the fire tanks via the screen wash system. These 2 systems do not intertie so this K/A is not applicable.
2 / 2	223001 / K6.10	Cooper Nuclear Station uses a Mark 1 containment, this K/A is for Mark 3 containments and does not apply.
2 / 2	223001 / A1.03	Cooper Nuclear Station uses a Mark 1 containment, this K/A is specific to Mark 3 containments and does not apply.
1 / 1	295019 / AK2.13	The subject K/A isn't relevant at the subject facility.
1 / 1	295027 / EA2.04	Cooper Nuclear Station uses a Mark 1 containment, this K/A is specific to a Mark 3 containment and does not apply.
2 / 2	202002 / A2.08	Cooper Nuclear Station is a BWR 4 and does not have Flow Control Valves in the Recirculation Flow control system, this K/A is specific to BWR 5, 6 and does not apply.
1 / 1	295038 / 2.4.49	K/A has no compatibility with 10CFR55.43 (b)
1 / 1	600000 / AA1.08	It isn't possible to prepare a psychometrically sound question related to the subject K/A.
2 / 1	209002 / 2.2.12	The subject K/A isn't relevant at the subject facility.
2 / 1	262002 / A2.04	The subject K/A isn't relevant at the subject facility.
2 / 1	203000 / K5.01	It isn't possible to prepare a psychometrically sound question related to the subject K/A.
2 / 2	215002 / 2.2.38	The subject K/A isn't relevant at the subject facility.
2 / 2	202001 / K3.06	The subject K/A isn't relevant at the subject facility.
2 / 2	201004 / A3.03	The subject K/A isn't relevant at the subject facility.