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CP- 201000424 TXX -10023 Ref. #

10CFR50.73(a)(2)(i)(B) 10CFR50.73(a)(2)(vii)

March 18, 2010

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

SUBJECT: COMANCHE PEAK NUCLEAR POWER PLANT DOCKET NOS. 50-445 and 50-446 LOSS OF AUTOMATIC INITIATION OF AUXILIARY FEEDWATER UPON LOSS OF MAIN FEEDWATER LICENSEE EVENT REPORT 445/10-002-00

Dear Sir or Madam:

Enclosed is Licensee Event Report (LER) 445/10-002-00, "Loss of Automatic Initiation of Auxiliary Feedwater Upon Loss of Main Feedwater," for Comanche Peak Nuclear Power Plant (CPNPP) Units 1 and 2.

This communication contains no licensing basis commitments regarding CPNPP Units 1 and 2.

Should you have any questions, please contact Gary Merka at (254) 897-6613.

Sincerely,

Luminant Generation Company LLC

Rafael Flores

By:

Fred W. Madden Director, Oversight & Regulatory Affairs

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NRR

A member of the STARS (Strategic Teaming and Resource Sharing) Alliance

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Callaway · Comanche Peak · Diablo Canyon · Palo Verde · San Onofre · South Texas Project · Wolf Creek

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Enclosure

c - E. E. Collins, Region IV B. K. Singal, NRR Resident Inspectors, Comanche Peak

NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSI (9-2007)	APPROVED BY OMB NO. 3150-0104 EXPIRES:					
LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)	Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of fundamagement and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.					
1. FACILITY NAME	2. DOCKET NUMBER 3. PAGE					
Comanche Peak Nuclear Power Plant Unit 1	05000 445 1 OF 5					
4. TITLE	· · · · · · · · · · · · · · · · · · ·					
Loss of Automatic Initiation of Auxiliary Feedwater Up						
5. EVENT DATE 6. LER NUMBER 7. REPORT						
MONTH DAY YEAR YEAR SECUENTIAL REV MONTH DAY	YEAR CPNPP Unit 2 Document Number					
01 20 2010 2010 002 00 03 18	2010 FACILITY NAME DOCUMENT NUMBER 05000					
20.2201(b) 20.2203(a)(3)(i) 3 20.2201(d) 20.2203(a)(3)(i) 20.2203(a)(1) 20.2203(a)(3)(i) 20.2203(a)(4) 20.2203(a)(2)(i) 50.36(c)(1)(i)(A) 10. POWER LEVEL 20.2203(a)(2)(i) 50.36(c)(1)(i)(A)	50.73(a)(2)(ii)(A) 50.73(a)(2)(viii)(A) 50.73(a)(2)(ii)(B) 50.73(a)(2)(viii)(B) 50.73(a)(2)(iii) 50.73(a)(2)(viii)(B) 50.73(a)(2)(iii) 50.73(a)(2)(viii)(A)					
20.2203(a)(2)(iii) 50.36(c)(2) 0 20.2203(a)(2)(iv) 50.46(a)(3)(ii) 20.2203(a)(2)(v) 50.73(a)(2)(i)(A) 20.2203(a)(2)(vi) X 50.73(a)(2)(i)(B)	50.73(a)(2)(v)(A) 73.71(a)(4) 50.73(a)(2)(v)(B) 73.71(a)(5) 50.73(a)(2)(v)(C) OTHER 50.73(a)(2)(v)(D) VOLUNTARY LER					
FACILITY NAME 12. LICENSEE CONTAG	T FOR THIS LER TELEPHONE NUMBER (Include Area Code)					
Timothy A. Hope, Nuclear Licensing Ma	, , , , , , , , , , , , , , , , , , , ,					
13. COMPLETE ONE LINE FOR EACH COMPONE						
CAUSE SYSTEM COMPONENT MANU- REPORTABLE CAU						
FACTURER TO EPIX	FACTURER TO EPIX					
	15. EXPECTED MONTH DAY YEAR					
YES (If yes, complete 15. EXPECTED SUBMISSION DATE)	SUBMISSION 05 31 2010					
ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)						
On January 20, 2010, Comanche Peak Nuclear Power Plant (CPNPP) Unit 1 was in Mode 3 during a planned outage and Unit 2 was in Mode 1 operating at 100% power. At 1827 hours, during review of an Oconee INPO OE report, parallels were discovered between the Oconee design and the CPNPP design. Further investigation determined that the required completion times for TS 3.3.2, Condition J, may not have been completed in the past when Units 1 and 2 were operating at low power with only one MFWP providing MFW flow.						

to ensure compliance with TS. Since Unit 2 has been operating at 100% power with both MFW pumps in service since discovery of this condition, corresponding changes to the Unit 2 procedures will be completed prior to removing one MFW pump from service during the next shutdown of Unit 2 or prior to startup of Unit 2 from any event which results in an unplanned trip of Unit 2.

All times in this report are approximate and Central Standard Time unless noted otherwise.

NRC FORM 366A (9-2007)	U.S. NUCLEAR REGULATORY COMMISSION
(3-2007)	LICENSEE EVENT REPORT (LER) CONTINUATION SHEET
Coman	1. FACILITY NAME 2. DOCKET 6. LER NUMBER 3. PAGE che Peak Nuclear Power Plant Unit 1 YEAR SEQUENTIAL REV NUMBER NO. 05000 – 445 2010 002 00 2 OF 5
NARRATIVE (If n	nore space is required, use additional copies of NRC Form 366A) (17)
I. DE	SCRIPTION OF THE REPORTABLE EVENT
А.	REPORTABLE EVENT CLASSIFICATION
	10CFR50.73(a)(2)(i)(B) "Any operation or condition which was prohibited by the plant's Technical Specifications" and 10CFR50.73(a)(2)(vii) "Any event where a single cause or condition caused at least one independent train or channel to become inoperable in multiple systems or two independent trains or channels to become inoperable in a single system designed to: (B) Remove residual heat"
В.	PLANT CONDITION PRIOR TO EVENT
	On January 20, 2010, CPNPP Unit 1 was in Mode 3 during a planned outage and Unit 2 was in Mode 1 operating at 100% power.
C.	STATUS OF STRUCTURES, SYSTEMS, OR COMPONENTS THAT WERE INOPERABLE AT THE START OF THE EVENT AND THAT CONTRIBUTED TO 'THE EVENT
	There were no inoperable structures, systems, or components that contributed to the event.
D.	NARRATIVE SUMMARY OF THE EVENT, INCLUDING DATES AND APPROXIMATE TIMES
	CPNPP TS 3.3.2, Function 6.g, requires two Main Feedwater pump (MFWP) trip channels for each automatic Auxiliary Feedwater Pump (AFW) initiation circuit. TS 3.3.2 also requires a manual initiation circuit, which was not affected by this event. Per TS 3.3.2, Condition J, if one required MFWP trip channel is inoperable, the inoperable trip channel must be placed in the trip condition within 6 hours or the plant must be placed in MODE 3 within the following 12 hours.
	On January 20, 2010, CPNPP Unit 1 was in Mode 3 during a planned outage and Unit 2 was in Mode 1 operating at 100% power. At 1827 hours, during review of Oconee INPO OE30225 ("Main Feedwater Pump Reset design feature provides incorrect Main Feedwater Pump status"), a System Engineer (Utility, Non-Licensed) discovered parallels between the Oconee design and the CPNPP design. Further investigation determined that CPNPP operating procedures allowed for one MFWP [EIIS: (SJ)(P)] to be reset (i.e., hydraulic oil pressure present) but not aligned to be a source of MFW.
	During low-power plant operations, such as start-up or when Unit 1 was recently operated at a lower power level while a single Main Transformer was in service, only one MFWP is creating forward MFW flow. The other MFWP is in a "reset" condition, with the Steam Stop Valves [EIIS: (SJ)(SHV)] open. This condition utilizes hydraulic trip fluid to keep the

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LICENSEE EVENT REPORT (LER)

CONTINUATION SHEET

1. FACILITY NAME	2. DOCKET	6. LER NUMBER		3. PAGE	
Comanche Peak Nuclear Power Plant Unit 1		YEAR	SEQUENTIAL NUMBER	REV NO	
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NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17)

Stop Valves open, which appears to be an "operating" MFWP to the hydraulic pressure switches [EIIS: (SJ)(HCU)(PS)]. In the event that the operating MFWP receives a "trip" signal, all of the MFW flow would cease, but since the off-line MFWP is in the "reset" condition, the 2/2 "Loss of Main Feedwater" logic would not be satisfied, and an AFW auto-start signal [EIIS: (BA)(MSTR)] would not be initiated. It was determined that in this condition, the MFWP trip channel would be inoperable per TS 3.3.2, Condition J. CPNPP believes that this condition has existed since 1990, when Unit 1 was initially licensed. Therefore, the required actions of TS 3.3.2, Condition J, may not have been complied with since 1990 at any time that CPNPP Units 1 and 2 were operating at low power with only one MFWP providing MFW flow.

E. THE METHOD OF DISCOVERY OF EACH COMPONENT OR SYSTEM FAILURE, OR PROCEDURAL PERSONNEL ERROR

During review of an Oconee INPO OE report, a System Engineer (Utility, Non-licensed) discovered parallels between the Oconee design and the CPNPP design. Further investigation determined that the required completion times for TS 3.3.2, Condition J, may not have been completed in the past within the times specified in the TS.

- II. COMPONENT OR SYSTEM FAILURES
 - A. CAUSE OF EACH COMPONENT OR SYSTEM FAILURE

Not applicable - No component failures were identified during this event.

B. FAILURE MODE, MECHANISM, AND EFFECTS OF EACH FAILED COMPONENT

Not applicable - No component failures were identified during this event.

C. SYSTEMS OR SECONDARY FUNCTIONS THAT WERE AFFECTED BY FAILURE OF COMPONENTS WITH MULTIPLE FUNCTIONS

Not applicable - No component failures were identified during this event.

D. FAILED COMPONENT INFORMATION

Not applicable - No component failures were identified during this event.

- III. ANALYSIS OF THE EVENT
 - A. SAFETY SYSTEM RESPONSES THAT OCCURRED
 - Not applicable No safety system responses occurred as a result of this event.

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(9-2007)	LICENSEE EVENT F CONTINUATIO)				
	1. FACILITY NAME	2. DOCKET	6	5. LER NUMBEI	२	3.	PAGE
Coman	che Peak Nuclear Power Plant Unit 1		YEAR	SEQUENTIAL NUMBER	REV NO.		
		05000 - 445	2010	002	00	4	OF 5
NARRATIVE (If r	nore space is required, use additional copies of NRC Form 3	366A) (1 7)			~		
В.	B. DURATION OF SAFETY SYSTEM TRAIN INOPERABILITY						
	The required actions of TS 3.3.2, Condit 1990 at any time that CPNPP Units 1 ar MFWP providing MFW flow.						
C.	SAFETY CONSEQUENCES AND IMPLIC	ATIONS OF 1	THE EVE	NT			
	The applicable safety function is decay h driven AFW pumps are designed to star level in any steam generator, upon act restoration of bus power following a loss of the safety analyses presented in FSAR C the motor-driven AFW pumps upon gen steam generator. Even though the TS r driven AFW pumps upon trip of both MI 1990 at any time that CPNPP Units 1 ar MFWP providing MFW flow, the AFW pur the motor-driven AFW pumps upon trip of the loss of normal decay heat removal; Chapter 15 accident analyses. Automatic still possible via steam generator low-low are credited within the safety analyses), Motor Driven AFW pumps), upon actuation signal after restoration of bus power follow also available via manual actuation by cor Based on the availability of the motor-driv analyses, the unavailability of the ant conclusions of the accident analyses.	t automaticall tuation of a 3 of offsite powe hapter 15, the neration of a equired auton FW pumps m and 2 were ope mps remained of both MFWF this trip fund cactuation of a water level pro- via AMSAC on of a Safety wing a loss of ntrol room ope	y in the Safety In er, or upo only cre low-low natic initia ay not h erating at l operabl Ps is an a ction is r the Moto obtection c (which s Injection offsite p rators as p start sig	event of a jection or in trip of bo edited signa water level ation circuit ave been of low power e. The aut anticipatory tot credited r Driven AF circuits (whi signal, and ower. AFW required by gnal credite	low-low AMSAC, th MFWF I is the s signal if of the r operable with onl omatic s trip bas I in the W pump ch, at CF ne Driver I by a bla V initiation y TS 3.3.	wate afte Ps. I tart on noto sinc y on tart o ed o FSA PNPF n an ockou n wa 2. safet	er er of yr- ee of nR so, d t s y
IV. CA	USE OF THE EVENT						
	e cause of this event is still being determ prmation will be submitted by May 31, 2010.		uppleme	ntal report	providin	g thi	is _.
v. co	RRECTIVE ACTIONS						
cor ser	rrective actions include the completion npliance with TS. Since Unit 2 has been vice since discovery of this condition, corre completed prior to removing one MFWP from	operating at sponding cha	100% po nges to f	wer with be the Unit 2 p	oth MFW	/Ps i əs w	in ill

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LICENSEE EVENT REPORT (LER) 1. FACILITY NAME 2. DOCKET 1. CACILITY NAME 3. PAGE Comanche Peak Nuclear Power Plant Unit 1 2. DOCKET 1. South and the second sec	NRC FORM 366A (9-2007)		U.S. NUCLEAR REGULATORY COMMISSION		
1. FACILITY NAME 2. DOCKET 6. LER NUMBER 3. PAGE Comanche Peak Nuclear Power Plant Unit 1 05000 – 445 2010 002 00 5 OF 5 NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17) prior to startup of Unit 2 from any event which results in an unplanned trip of Unit 2. An Operations Shift Order has been issued to advise operating staff of the condition and to note that the Unit 2 procedures will need to be updated/revised prior to a restart of Unit 2 should a trip or shutdown occur and a restart is required. This shift order item will remain in effect until the Unit 2 procedures are updated. VI. PREVIOUS SIMILAR EVENTS	LICENSEE EVENT REPORT (LER)				
 NARRATIVE (If more space is required, use additional copies of NRC Form 366A) (17) prior to startup of Unit 2 from any event which results in an unplanned trip of Unit 2. An Operations Shift Order has been issued to advise operating staff of the condition and to note that the Unit 2 procedures will need to be updated/revised prior to a restart of Unit 2 should a trip or shutdown occur and a restart is required. This shift order item will remain in effect until the Unit 2 procedures are updated. VI. PREVIOUS SIMILAR EVENTS 	1. FACILITY NAME	2. DOCKET	YEAR SEQUENTIAL REV NUMBER NO.		
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	Operations Shift Order has been issued t that the Unit 2 procedures will need to be trip or shutdown occur and a restart is red	o advise operating updated/revised p	g staff of the condition and to note prior to a restart of Unit 2 should a		
There have been no previous similar reportable events at CPNPP in the last three years.	VI. PREVIOUS SIMILAR EVENTS				
	There have been no previous similar repo	rtable events at CF	PNPP in the last three years.		
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