

June 30, 2010

APPLICANT: MITSUBISHI HEAVY INDUSTRIES, LTD.

APPLICATION: US-APWR STANDARD DESIGN CERTIFICATION REVIEW

SUBJECT: SUMMARY OF THE FEBRUARY 3, 2010, MEETING WITH MITSUBISHI HEAVY INDUSTRIES LTD. TO DISCUSS SAFETY INSTRUMENTATION AND CONTROLS DESCRIPTION AND DESIGN PROCESS TOPICAL REPORT, SAFETY SYSTEMS DIGITAL PLATFORM - MELTAC TOPICAL REPORT AND THE UNITED STATES - ADVANCED PRESSURIZED WATER REACTOR DESIGN CONTROL DOCUMENT CHAPTER 7

On February 3, 2010, a Category 1 public meeting was held between the U.S. Nuclear Regulatory Commission (NRC), Mitsubishi Heavy Industries, Ltd. (MHI), Mitsubishi Nuclear Energy Systems, Mitsubishi Electric Power Products, Inc., Mitsubishi Electric Corporation representatives and members of the Public. Enclosed are the Meeting Minutes and Presentation, which include more detailed discussions of the Safety Instrumentation and Control (I&C) Topical Report, the Mitsubishi Electric Total Advanced Controller (MELTAC) Topical Report as well as the Chapter 7 of the United States - Advanced Pressurized Water Reactor (US-APWR) Design Control Document (DCD), and supporting documents.

The NRC staff opened discussions by stating that the Office of Nuclear Reactor Regulation (NRR) will discontinue review of the Safety I&C Topical Report for Operating Reactors because the topical report doesn't contain sufficient information to make a safety determination. MHI accepted this position. The NRC staff also presented technical issues with the Safety I&C Topical Report, including safety functions, priority logic, protection actions, engineering tool, quality assurance, and supporting documents.

Since the MELTAC Topical Report is no longer being reviewed by NRR, the NRC staff requested that references to operating reactors should be removed. In addition, the NRC staff followed up with discussions concerning standards, engineering tool connection and communication, digital communications, quality assurance, Commercial Grade Dedication, and conformance to Branch Technical Position (BTP) 7-14. MHI stated that they would revise the MELTAC Topical Report to address the NRC staff's comments and resubmit the topical report. MHI also stated that they would submit the MELTAC Software Program Manual for the basic software by the end of April, 2010, to address the guidelines of BTP 7-14.

The NRC staff also presented issues with supporting documents for the Safety I&C Topical Report, MELTAC Topical Report, and US-APWR DCD Chapter 7 review. Comments focused on proper referencing, timeline of submission, identification of standards, and technical clarifications. MHI took away several action items to address the NRC's comments, including revision and resubmission of supporting documents.

Members of the public were in attendance and participated via a conference call bridge. No comments were received from the public. Please direct any inquiries to Michael S. Magee at (301) 415-6988, or via e-mail at Michael.Magee@nrc.gov.

/RA/

Michael S. Magee, Project Manager
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Division of New Reactor Licensing
Office of New Reactors

Docket No. 52-021

Enclosures:

1. List of Attendees
2. Meeting Minutes (ML101380108)
3. Meeting Presentation (ML100600438)

cc w/encl: See next page

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