UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 25, 2010

Mr. Larry Meyer
Site Vice President
NextEra Energy Point Beach, LLC
6610 Nuclear Road
Two Rivers, WI 54241-9516

SUBJECT: POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2 - REQUEST FOR ADDITIONAL INFORMATION FROM ELECTRICAL ENGINEERING BRANCH RE: EXTENDED POWER UPRATE (TAC NOS. ME1044 AND ME1045)

Dear Mr. Meyer:

By letter to the U.S. Nuclear Regulatory Commission (NRC) dated April 7, 2009, as supplemented by letters dated September 11 and October 9, 2009 (Agencywide Documents Access and Management System Accession Nos. ML091250564, ML092570205, and ML092860098), FPL Energy Point Beach, LLC, submitted a request to increase each unit's licensed core power level from 1540 megawatts thermal (MWt) to 1800 MWt reactor core power, and revise the technical specifications to support operation at this increased core thermal power level.

The NRC staff is reviewing your submittal and has determined that additional information is required to complete the review. The specific information requested is addressed in the enclosure to this letter. During a discussion with your staff on March 22, 2010, it was agreed that you would provide the additional information within 30 days of the date of this letter.

The NRC staff considers that timely responses to requests for additional information help ensure sufficient time is available for staff review and contribute toward the NRC's goal of efficient and effective use of staff resources. If circumstances result in the need to revise the requested response date, please contact me at (301) 415-2048.

Sincerely,

Justin C. Poole, Project Manager
Plant Licensing Branch III-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-266 and 50-301

Enclosure:
Request for Additional Information

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REQUEST FOR ADDITIONAL INFORMATION

POINT BEACH NUCLEAR POWER PLANT, UNITS 1 AND 2

DOCKET NOS. 50-266 AND 50-301

The proposed change would extend the time delay of the loss of voltage (LOV) relay to 2.3 seconds when it actuates at approximately 3156V. The proposed change also extends the closure time for emergency diesel generator (EDG) breaker to 3.55 seconds. The degraded voltage (DV) relay is set at approximately 3937V with a time delay of 5.68 seconds with a concurrent safety injection signal (SIS) and 39.14 seconds without a SIS available. The DV protection scheme does not directly start the EDG. A trip of the DV relay actuates the LOV relay which adds 2.3 seconds for relay actuation and 3.55 seconds for breaker closure, to the total time for start of EDG.

EEEB-1: Provide a summary of the analyses that demonstrates that the safety related equipment would not be degraded when required to operate at a voltage level marginally above the LOV relay setpoint for approximately 39 seconds.

EEEB-2: Provide a summary of the analyses that demonstrates that plant safety is not compromised if EDG start is delayed by approximately 42 seconds and EDG breaker closure is delayed by an additional 3.55 seconds following degraded grid conditions with plant bus voltages marginally above the LOV relay setting for 39 seconds.

EEEB-3: The undervoltage protective schemes (LOV and DV) are not bypassed during EDG A operation. Point Beach Technical Specifications do not stipulate transient or steady state EDG voltage requirements. Provide details on acceptance criteria established in plant procedures to ensure that EDG achieves acceptable voltage band within the allowed time during an emergency start. Provide a comparison of the LOV and DV relay reset values and the criteria for acceptable EDG voltage.

EEEB-4: Plant licensing basis assumes simultaneous loss of offsite power (LOOP) coupled with a design basis accident. For events such as a Large Break Loss of Coolant Accident, a SIS starts the EDG at the onset of the event. For other events that may not generate a SIS immediately (such as steam generator tube rupture) occurring simultaneously with a LOOP, provide details on consequences of delaying EDG start by 2.3 seconds and adding 3.55 seconds for breaker closure when the plant is operating at uprated conditions.

The following questions pertain to High Energy Line Break Environmental Qualification (EQ) of the equipment or components.

EEEB-5: The NRC staff requests the licensee to provide a detailed comparison of the following EQ temperature and pressure profiles:

a. Current conditions
b. Existing bounding EQ profile
c. At proposed EPU conditions

EEEB-6: The NRC staff requests the licensee to provide a detailed discussion regarding whether the existing EQ profiles envelop the proposed EPU conditions.

EEEB-7: The NRC staff requests the licensee to provide radiation doses for the inside and outside containment due to the EPU condition. Provide a discussion and confirm that the EQ components are still qualified.

EEEB-8: The NRC staff requests the licensee to identify the existing components that are being replaced due to the EPU conditions and confirm that replacements are qualified in accordance with Title 10 of the *Code of Federal Regulations*, Section 50.49.

EEEB-9: The NRC staff requests the licensee to identify any new components added to the EQ program due to the EPU conditions. Furthermore, the staff requests the licensee to confirm that these components, if of the same model or make, as components already in the EQ program, are also maintained per the EQ program.

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/RA/

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ADAMS Accession Number: ML100810368 *per memos dated December 15, 2009 and March 3, 2010

OFFICE	LPL3-1/PM	LPL3-1/LA	NRR/EEEB/BC	LPL3-1/BC
NAME	JPoole	BTully	GWilson*	RPascarelli
DATE	03/25/10	03/23/10	12/15/09 and 3/3/10	03/25/10

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