UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION IV
612 EAST LAMAR BLVD, SUITE 400
ARLINGTON, TEXAS 76011-4125

February 24, 2010

Rafael Flores, Senior Vice President
and Chief Nuclear Officer
Attention: Regulatory Affairs
Luminant Generation Company LLC
P.O. Box 1002
Glen Rose, TX 76043

SUBJECT: Comanche Peak Steam Electric Station – NOTIFICATION OF INSPECTION (NRC INSPECTION REPORT 05000445/2010003) AND REQUEST FOR INFORMATION

Dear Mr. Flores:

From April 5-9 and April 19-23, 2010, reactor inspectors from the Nuclear Regulatory Commission's (NRC) Region IV office will perform the baseline inservice inspection at Comanche Peak Steam Electric Station, Unit 1, using NRC Inspection Procedure 71111.08, "Inservice Inspection Activities," and TI 2515/172, "Reactor Coolant System Dissimilar Metal Butt Welds." Experience has shown that this inspection is a resource intensive inspection both for the NRC inspectors and your staff. In order to minimize the impact to your onsite resources and to ensure a productive inspection, we have enclosed a request for documents needed for this inspection. These documents have been divided into two groups. The first group (Section A of the enclosure) identified information to be provided prior to the inspection to ensure that the inspectors are adequately prepared. The second group (Section B of the enclosure) identifies the information the inspectors will need upon arrival at the site. It is important that all of these documents are up to date and complete in order to minimize the number of additional documents requested during the preparation and/or the onsite portions of the inspection.

We have discussed the schedule for these inspection activities with your staff and understand that our regulatory contact for this inspection will be Mr. Barnet of your licensing organization. Our inspection dates are subject to change based on your updated schedule of outage activities. If there are any questions about this inspection or the material requested, please contact the lead inspector Wayne Sifre at (817) 860-8193 (wayne.sifre@nrc.gov) or Matthew Young at (817) 276-6545 (matt.young@nrc.gov).

This letter does not contain new or amended information collection requirements subject to the Paperwork Reduction Act of 1995 (44 U.S.C. 3501 et seq.). Existing information collection

requirements were approved by the Office of Management and Budget, Control Number 3150-0011. The NRC may not conduct or sponsor, and a person is not required to respond to, a request for information or an information collection requirement unless the requesting document displays a currently valid Office of Management and Budget control number.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC web site at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

Sincerely,

/RA/

Gregory E. Werner, Chief
Plant Support Branch 2
Division of Reactor Safety

Docket: 50-445
License: NPF-87

Enclosure:
Inservice Inspection Document Request

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OEMail Resource

Inspection Reports/MidCycle and EOC Letters to the following:
ROPreports

Only inspection reports to the following:
DRS/TSB STA (Dale.Powers@nrc.gov)
OEDO RIV Coordinator (Leigh.Trocine@nrc.gov)

SUNSI Review Completed: __Y__ ADAMS: ☒ Yes □ No Initials: ____
☒ Publicly Available □ Non-Publicly Available □ Sensitive ☒ Non-Sensitive

R:\REACTORS\CP U1 ISI RFI 2010 - WCS.

SRI:EB1	C:PSB2			
W. Sifre	G. Werner			
/RA/	/RA/			
2/18/10	2/24/10			

OFFICIAL RECORD COPY T=Telephone E=E-mail F=Fax

INSERVICE INSPECTION DOCUMENT REQUEST

Inspection Dates: April 5-9 and April 19-23, 2010

Inspection Procedures: IP 71111.08 "Inservice Inspection (ISI) Activities"
TI 2515/172 "Reactor Coolant System Dissimilar Metal Butt Welds"

Inspectors: Wayne Sifre, Senior Reactor Inspector (Lead Inspector - ISI)
Matthew Young, Reactor Inspector

A. Information Requested for the In-Office Preparation Week

The following information should be sent to the Region IV office in hard copy or electronic format (ims.certrec.com preferred), in care of Wayne Sifre, by March 22, 2010, to facilitate the selection of specific items that will be reviewed during the onsite inspection week. The inspectors will select specific items from the information requested below and then request from your staff additional documents needed during the onsite inspection week (Section B of this enclosure). We ask that the specific items selected from the lists be available and ready for review on the first day of inspection. *Please provide requested documentation electronically if possible. If requested documents are large and only hard copy formats are available, please inform the inspectors, and provide subject documentation during the first day of the onsite inspection. If you have any questions regarding this information request, please call the inspectors as soon as possible.

A.1 ISI/Welding Programs and Schedule Information

a) A detailed schedule (including preliminary dates) of:

i) Nondestructive examinations (NDEs) planned for Class 1 & 2 systems and containment, performed as part of your ASME Section XI, Risk Informed (if applicable), and augmented ISI programs during the upcoming outage.

Provide a status summary of the NDE inspection activities vs. the required inspection period percentages for this interval by category per ASME Section XI, IWX-2400 (Do not provide separately if other documentation requested contains this information).

ii) Reactor pressure vessel head examinations planned for the upcoming outage.

iii) Examinations planned for Alloy 82/182/600 components that are not included in the Section XI scope (If applicable).

iv) Examinations planned as part of your Boric Acid Corrosion Control Program (Mode 3 walkdowns, bolted connection walkdowns, etc.).

v) Welding activities that are scheduled to be completed during the upcoming outage (ASME Class 1, 2, or 3 structures, systems, or components (SSCs)).

b) A copy of ASME Section XI Code Relief Requests and associated NRC Safety Evaluations applicable to the examinations identified above.

c) A list of NDE reports (ultrasonic, radiography, magnetic particle, dye penetrate, Visual VT-1, VT-2, and VT-3), which have identified relevant conditions on Code Class 1 & 2 systems since the beginning of the last refueling outage. This should include the previous Section XI pressure tests conducted during start up and any evaluations associated with the results of the pressure tests. Also, include in the list the NDE reports with relevant conditions in the reactor pressure vessel head penetration nozzles which have been accepted for continued service. The list of NDE reports should include a brief description of the SSC where the relevant condition was identified.

d) A list with a brief description (e.g., system, material, pipe size, weld number, and NDE performed) of the welds in Code Class 1 and 2 systems which have been fabricated due to component repair/replacement activities since the beginning of the last refueling outage or are planned to be fabricated this refueling outage.

e) If reactor vessel weld examinations required by the ASME Code are scheduled to occur during the upcoming outage, provide a detailed description of the welds to be examined and the extent of the planned examination. Please also provide reference numbers for applicable procedures that will be used to conduct these examinations.

f) Copy of any 10 CFR Part 21 reports applicable to your SSCs within the scope of Section XI of the ASME Code that have been identified since the beginning of the last refueling outage.

g) A list of any temporary noncode repairs in service (e.g., pinhole leaks).

h) Please provide copies of the most recent self assessments for the ISI, Welding, and Alloy 600 Programs.

A.2 Reactor Pressure Vessel Head

a) Provide the detailed scope of the planned NDE of the reactor vessel head which identifies the types of NDE methods to be used on each specific part of the vessel head to fulfill commitments made in response to NRC Bulletin 2002-02 and NRC Order EA-03-009. Also, include examination scope expansion criteria and planned expansion sample sizes if relevant conditions are identified (if applicable).

b) A list of the standards and/or requirements that will be used to evaluate indications identified during NDE of the reactor vessel head (e.g., the specific

industry or procedural standards which will be used to evaluate potential leakage and/or flaw indications).

A.3 Boric Acid Corrosion Control Program

a) Copy of the procedures that govern the scope, equipment and implementation of the inspections required to identify boric acid leakage and the procedures for boric acid leakage/corrosion evaluation.

b) Please provide a list of leaks (including Code class of the components) that have been identified since the last refueling outage and associated corrective action documentation. If during the last cycle, the Unit was shutdown, please provide documentation of containment walkdown inspections performed as part of the Boric Acid Corrosion Control Program.

c) Please provide a copy of the most recent self-assessment performed for the Boric Acid Corrosion Control Program.

A.4 Materials Reliability (MRP-139) Program Activities

a) A list of, with verification, that the baseline inspections of all applicable Dissimilar Metal Butt Welds (DMBWs) have been completed by December 31, 2009.

b) Verification that baseline inspection of hot leg and cold leg temperature DMBWs have been included in the inspection program and that the schedules for the baseline inspections are consistent with the baseline schedules in MRP-139.

c) A list and schedule (examination dates) of all DMBW examinations planned for the upcoming refueling outage. If none are scheduled, then data from previous examinations should be available for review.

d) A list and schedule for any welding to be performed on DMBWs in the upcoming outage. If no welding will be performed then records of previous welding on DMBWs and postweld NDE documentation.

e) A list and schedule of any stress improvement activities planned for the upcoming refueling outage. If none are scheduled, then have qualification reports for any stress improvements previously performed.

f) Documentation and description of how the baseline and ISI specifications in MRP-139 are satisfied at your facility.

g) A list of weld locations controlled by MRP-139 which includes the MRP-139 weld category, pre-mitigation volumetric examination date, type of mitigation or stress improvement, post mitigation volumetric examination date, and plans for future MRP-139 required examinations.

A.6 Additional information related to all ISI activities

a) A list with a brief description of ISI, Boric Acid Corrosion Control Program, and steam generator tube inspection related issues (e.g., condition reports) entered into your corrective action program since the beginning of the last refueling outage (for Unit 1). For example, a list based upon data base searches using key words related to piping or steam generator tube degradation such as: ISI, ASME Code, Section XI, NDE, cracks, wear, thinning, leakage, rust, corrosion, boric acid, or errors in piping/steam generator tube examinations.

b) Please provide names and phone numbers for the following program leads:

ISI contacts (Examination, planning)
Containment Exams
Reactor Pressure Vessel Head Exams
Snubbers and Supports
Repair and Replacement Program Manager
Licensing Contact
Site Welding Engineer
Boric Acid Corrosion Control Program

B. Information to be provided onsite to the inspectors at the entrance meeting (April 5, 2010):

B.1 ISI/Welding Programs and Schedule Information

a) Updated schedules for ISI/NDE activities, including steam generator tube inspections, planned welding activities, and schedule showing contingency repair plans, if available.

b) For ASME Code Class 1 and 2 welds selected by the inspectors from the lists provided from Section A of this enclosure, please provide copies of the following documentation for each subject weld:

i) Weld data sheet (traveler)

ii) Weld configuration and system location

iii) Applicable Code Edition and Addenda for weldment

iv) Applicable Code Edition and Addenda for welding procedures

v) Applicable welding procedures used to fabricate the welds

vi) Copies of procedure qualification records supporting the welding procedures from B.1.b.v

vii) Copies of mechanical test reports identified in the procedure qualification records above

viii) Copies of the nonconformance reports for the selected welds (if applicable)

ix) Radiographs of the selected welds and access to equipment to allow viewing radiographs (if RT was performed)

x) Copies of the preservice examination records for the selected welds

xi) Copies of welder performance qualifications records applicable to the selected welds, including documentation that welder maintained proficiency in the applicable welding processes specified in the weld procedures (at least 6 months prior to the date of subject work)

xii) Copies of NDE personnel qualifications (VT, PT, UT, RT), as applicable

c) For the ISI related corrective action issues selected by the inspectors from Section A of this enclosure, provide a copy of the corrective actions and supporting documentation.

d) For the NDE reports with relevant conditions on Code Class 1 & 2 systems selected by the inspectors from Section A above, provide a copy of the examination records, examiner qualification records, and associated corrective action documents.

e) A copy of (or ready access to) most current revision of the ISI Program Manual and Plan for the current interval.

f) For the activities selected by the inspectors from Section A of this enclosure, provide copy of the NDE procedures used to perform the examinations (including calibration and flaw characterization/sizing procedures). For ultrasonic examination procedures qualified in accordance with ASME Code, Section XI, Appendix VIII, provide documentation supporting the procedure qualification (e.g., the EPRI performance demonstration qualification summary sheets). Also, include qualification documentation of the specific equipment to be used (e.g., ultrasonic unit, cables, and transducers including serial numbers) and NDE personnel qualification records.

B.2 Reactor Pressure Vessel Head

a) Provide the NDE personnel qualification records for the examiners who will perform examinations of the reactor pressure vessel head.

b) Provide drawings showing the following (if a visual examination is planned for the upcoming refueling outage):

i) Reactor pressure vessel head and control rod drive mechanism nozzle configurations

ii) Reactor pressure vessel head insulation configuration

The drawings listed above should include fabrication drawings for the nozzle attachment welds as applicable.

c) Copy of NDE reports from the last reactor pressure vessel head examination.

d) Copy of evaluation or calculation demonstrating that the scope of the visual examination of the upper head will meet the 95 percent minimum coverage required by NRC Order EA-03-009 (if a visual examination is planned for the upcoming refueling outage).

e) Provide a copy of the procedures that will be used to identify the source of any boric acid deposits identified on the reactor pressure vessel head. If no explicit procedures exist which govern this activity, provide a description of the process to be followed including personnel responsibilities and expectations.

f) Provide a copy of the updated calculation of effective degradation years for the reactor pressure vessel head susceptibility ranking.

g) Provide copy of the vendor qualification reports that demonstrates the detection capability of the NDE equipment used for the reactor pressure vessel head examinations. Also, identify any changes in equipment configurations used for the reactor pressure vessel head examinations which differ from that used in the vendor qualification reports.

B.3 Boric Acid Corrosion Control Program

a) Please provide boric acid walkdown inspection results, an updated list of boric acid leaks identified so far this outage, associated corrective action documentation, and overall status of planned boric acid inspections.

b) Please provide any engineering evaluations completed for boric acid leaks identified since the end of the last refueling outage. Please include a status of corrective actions to repair and/or clean these boric acid leaks. Please identify specifically which known leaks, if any, have remained in service or will remain in service as active leaks.

B.4 Codes and Standards

a) Ready access to (i.e., copies provided to the inspectors for use during the inspection at the onsite inspection location, or room number and location where available):

i) Applicable editions of the ASME Code (Sections V, IX, and XI) for the ISI program and the repair/replacement program

ii) EPRI and industry standards referenced in the procedures used to perform the steam generator tube eddy current examination

Inspector Contact Information:

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