

EFFLUENT AND WASTE DISPOSAL

SEMI-ANNUAL REPORT

SECOND HALF - 1984

REVISED PAGES

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TABLE 3

EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT 1984  
 SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

A. SOLID WASTE SHIPPED OFFSITE FOR BURIAL OR DISPOSAL (Not irradiated fuel)

1. Type of waste	Unit	6-month Period	Est. Total Error, %
a. Spent resins, filter sludges, evaporator bottoms, etc.	m <sup>3</sup>	8.39 E 1	
	Ci	2.75 E 2	1.00 E 2
b. Dry compressible waste, contaminated equip, etc.	m <sup>3</sup>	5.87 E 2	
	Ci	3.11 E 0	1.00 E 2
c. Irradiated components, control rods, etc.	m <sup>3</sup>	. E	
	Ci	. E	. E
d. Other (describe)	m <sup>3</sup>	. E	
	Ci	. E	. E

2. Estimate of major nuclide composition (by type of waste)

a.	CS137	%	3.9 E+1
	CS134	%	1.4 E+1
	CO60	%	1.9 E+1
b.	CO58	%	0.7 E+1
	FE55	%	1.0 E+1
	NI63	%	0.9 E+1
c.	CO60	%	4.5 E+1
	CS137	%	0.7 E+1
	NI63	%	3.4 E+1
d.	FE55	%	0.6 E+1
		%	. E
		%	. E

3. Solid Waste Disposition

<u>Number of Shipments</u>	<u>Mode of Transportation</u>	<u>Destination</u>
32	Truck	Barnwell, SC
8	Truck	Richland, WA

B. IRRADIATED FUEL SHIPMENTS (Disposition)

<u>Number of Shipments</u>	<u>Mode of Transportation</u>	<u>Destination</u>
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## Reportable Changes

### A. Process Control Program

For the period reported there were no changes made to the Process Control Program.

### B. Offsite Dose Calculation Manual

- 1) The original ODCM approved by the NRC in their letter dated June 6, 1984 used an extreme conservatism factor of .157 in the calculation of the Noble Gas release rate. Subsequently, a more appropriate conservatism value of .8 was used consistent with of the section of the ODCM.
- 2) The listing titled "Indian Point Station - Location of Sampling Station Points" and attendant maps identifying the sample location were revised to be consistent with the Radiological Effluent Technical Specifications (RETS) of Appendix A to DPR-26 and to reflect the sample designation included in the RETS.

### C. Radioactive Waste Systems

For the period reported there were no major changes made to any of the radioactive waste systems.

The changes described above have been reviewed and found acceptable by the Station Nuclear Safety Committee. The Committee concurs that the changes will not reduce the accuracy or reliability of dose calculation or set point determination.

## Reportable Items

### A. Radioactive Liquid Effluent Monitoring Instrumentation

The steam generator blowdown effluent line flow rate measurement device has been inoperable for greater than 30 days and is being reported pursuant to Indian Point 2 Technical Specification 3.9.A.2.C. Prior to the issuance at the radiological effluent technical specifications, Amendment 90 to Indian Point Facility Operating License No. DPR-26 on June 20, 1984, the provisions to measure steam generator blowdown flow had been disconnected to establish sampling connections outside containment. Engineering has been underway to provide this capability. This new flow measurement equipment is expected to be operable by the end of the 1987 Refueling Outage.