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February 22, 2010

Michael Perito
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5485 US Highway 61N
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Subject: ERRATA FOR RIVER BEND STATION - NRC INTEGRATED INSPECTION
REPORT 05000458/2009005

Dear Mr. Perito:

This errata corrects administrative errors that were in the original report. Please replace page 3 of the Summary of Findings, pages 29-31 of the Report Details, and pages A-2 through A-15 of the Supplemental Information in NRC Inspection Report 05000458/2009005, dated February 9, 2010, with the enclosed revised pages.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, and its enclosure, will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Vincent G. Gaddy, Chief
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Docket: 50-458
License: NPF-47

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2/22/10	2/22/10				

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Therefore, the event in the worst case would have been transparent to the core. Also, the displaced inventory posed no threat to any of the plant's mitigating systems. The inspectors concluded that the finding was of very low safety significance (Green). There is no crosscutting aspect associated with this violation because the finding does not reflect current licensee performance (Section 1R22).

Cornerstone: Public Radiation Safety

- Green. The inspectors identified a noncited violation of 10 CFR 71.5 and 49 CFR 172.604(a) for a failure to ensure that the shift manager, whose phone number was listed as the required 24-hour emergency phone number on shipping documents, was knowledgeable about the radioactive waste shipment that left site on December 16, 2009, and had immediate access to a person who had specific information on the shipment. Specifically, the shift manager was listed as the required 24-hour contact; however, the shift managers (on multiple shifts) were not provided with documentation or information about the shipments that left the site on December 16, 2009. Although the shift manager would have eventually contacted a knowledgeable person, this delay would not have resulted in immediate access to the person with information related to the shipment. The licensee immediately provided the shift manager a copy of the shipping documentation, briefed the shift manager, and entered this issue into their corrective action program as Condition Report CR-RBS-2009-06419.

This performance deficiency was more than minor because it adversely affected the public radiation safety cornerstone to ensure adequate protection of public health and safety from exposure to radioactive materials released into the public domain in that the failure to have shipment information immediately available could restrict the actions of fire department and/or rescue personnel responding to an accident. When processed through the Public Radiation Safety Determination Process, the finding was determined to be of very low safety significance because the finding: (1) was associated with radioactive material control, (2) involved the licensee's program for radioactive material transportation, (3) did not cause radiation limits to be exceeded, (4) did not involve a breach of package during transit, (5) did not involve a certificate of compliance finding, (6) did not involve a low level burial ground nonconformance, and (7) did not involve a failure to make notifications. The inspectors determined the finding had a crosscutting aspect in area of resources, associated with documentation, because the licensee's procedures did not provide guidance on informing the control room about shipments and thus, the procedures were not complete, accurate nor up-to-date [H.2(c)] (Section 2PS2).

.4 Selected Issue Follow-up Inspection

a. Inspection Scope

During a review of items entered in the licensee's corrective action program, the inspectors recognized a corrective action item documenting operator attempt to shift the

reactor recirculation pumps from fast speed to slow during a plant shutdown for a refueling outage.

These activities constitute completion of one in-depth problem identification and resolution sample as defined in Inspection Procedure 71152-05.

b. Findings

No findings of significance were identified.

.5 In-depth Review of Operator Workarounds

a. Inspection Scope

An operator workaround is defined as a degraded or nonconforming condition that complicates the operation of plant equipment and is compensated for by operator action. During the week of November 2, 2009, the inspectors reviewed the cumulative effect of the existing operator workarounds and contingency plans. The inspectors concentrated on the effect the workarounds have on: (1) the reliability, availability, and potential for misoperation of any mitigating system; (2) whether they could increase the frequency of an initiating event; and (3) their effect on the operation of multiple mitigating systems. In addition, the inspectors reviewed the cumulative effects the operator workarounds have on the ability of the operators to respond in a correct and timely manner to plant transients and accidents.

These activities constitute completion of one in-depth review of operator workarounds sample as defined in Inspection Procedure 71152-05.

b. Findings

No findings of significance were identified.

40A3 Event Follow-up (71153)

.1 (Closed) Licensee Event Report 05000458/2009-002-00: Unplanned Manual Reactor Scram Following Trip of Both Reactor Recirculation Pumps

On September 20, 2009, plant operators were reducing reactor power to begin a refueling outage. At 37 percent reactor power, the operators attempted to shift both reactor recirculation pumps from fast to slow speed. Reactor recirculation pump A shifted to slow speed, but tripped off approximately nine seconds later. Reactor recirculation pump B tripped off directly from high speed. The operator's reactivity manipulation plan relied on forced circulation for plant shutdown. Without a plan for reactor shut down in the natural circulation mode, the operators inserted an unplanned controlled manual reactor scram. Thermal-hydraulic conditions remained within operational limits during the event. The licensee entered the event into their corrective action program as Condition Report CR-RBS-2009-04175. The inspectors discussed the event with licensee management, engineering, operations, and maintenance

personnel to understand the conditions leading to the loss of both reactor recirculation pumps and subsequent operator actions. This licensee event report is closed.

.2 (Closed) Licensee Event Report 05000458/2009-003-00: Low Pressure Coolant Injection Actuation Following Loss of Water from Upper Reactor Cavity Pool

The safety significance and the enforcement aspect of this violation are discussed in section 1R22. The licensee has entered the event into their corrective action program as Condition Report CR-RBS-2009-04681. This licensee event report is closed.

4OA5 Other Activities

Quarterly Resident Inspector Observations of Security Personnel and Activities

a. Inspection Scope

During the inspection period, the inspectors performed observations of security force personnel and activities to ensure that the activities were consistent with River Bend security procedures and regulatory requirements relating to nuclear plant security. These observations took place during both normal and off-normal plant working hours.

These quarterly resident inspector observations of security force personnel and activities did not constitute any additional inspection samples. Rather, they were considered an integral part of the inspectors' normal plant status review and inspection activities.

b. Findings

No findings of significance were identified.

4OA6 Meetings

Exit Meeting Summary

On October 2, 2009, the inspector presented the in-service inspection activities inspection results to Mr. J. Roberts, and other members of the licensee staff. The licensee acknowledged the issues presented. The inspector asked the licensee whether any materials examined during the inspection should be considered proprietary. No proprietary information was identified.

On December 17, 2009, the inspectors presented the radiation safety inspection results to Mr. M. Perito, Site Vice President, Operations, and other members of the licensee staff. The licensee acknowledged the issues presented. The inspectors asked the licensee whether any materials examined during the inspection should be considered proprietary. No proprietary information was identified.

On January 7, 2010, the inspectors presented the integrated baseline inspection results to Mr. E. Olson, General Manager, Plant Operations, and other members of the licensee staff. The licensee acknowledged the issues presented. The inspectors asked the licensee whether any

materials examined during the inspection should be considered proprietary. No proprietary information was identified.

40A7 Licensee-Identified Violations

The following violation of very low safety significance (Green) was identified by the licensee and is a violation of NRC requirements which meets the criteria of Section VI of the NRC Enforcement Policy, NUREG-1600, for being dispositioned as a noncited violation.

- Technical Specification 5.4.1.a requires implementation of applicable procedures recommended in Regulatory Guide 1.33, Revision 2, Appendix A, February 1978. Section 7(e) of the Appendix requires procedures for training in radiation protection and personnel monitoring. Section 4.0[1] of Procedure EN-RP-100, "Radworker Expectations," Revision 0, requires, in part, that workers notify health physics upon receipt of any electronic alarming dosimeter alarm that was not prebriefed. Contrary to this, on January 17, 2008, a licensee worker failed to notify radiation protection personnel upon receipt of an electronic alarming dosimeter dose rate alarm in steam jet air ejector room B that was not prebriefed. This finding had very low safety significance (Green) because: (1) it was not an ALARA finding, (2) there was no overexposure, (3) there was no substantial potential for an overexposure, and (4) the ability to assess dose was not compromised. This item was entered into the licensee's corrective action program as Condition Report CR-RBS-2008-00506.

LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED

Opened and Closed

05000458/2009005-01	NCV	Main Steam Line Plug Seal Failure Results in Loss of Reactor Cavity Inventory (Section 1R22)
05000458/2009005-02	NCV	Failure to Ensure the Emergency Contact had Knowledge About a Shipment (Section 2PS2)

Closed

05000458/2009-002-00	LER	Unplanned Manual Reactor Scram Following Trip of Both Reactor Recirculation Pumps (Section 4OA3.1)
05000458/2009-003-00	LER	Low Pressure Coolant Injection Actuation Following Loss of Water from Upper Reactor Cavity Pool (Section 4OA3.2)

LIST OF DOCUMENTS REVIEWED

Section 1R01: Adverse Weather Protection

MISCELLANEOUS

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION/DATE</u>
NUREG-1407	Procedural and Submittal Guidance for the Individual Plant Examination of External Events (IPEEE) for Severe Weather Vulnerabilities	0
NUREG-1779	Regulatory Effectiveness of the Station Blackout Rule	0
RIS-2004-05	Grid Reliability and the Impact on Plant Risk and the Operability of Offsite Power	April 15, 2004

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
AOP-0029	Severe Weather Operation	21
AOP-0064	Degraded Grid	0
EN-LI-101	10CFR50.59 Review Program	4

ENS-DC-199	Off-Site Power Supply Design Requirements	2
ENS-DC-201	ENS Transmission Grid Monitoring	2
OSP-0063	Grid Monitor	1

Section 1R04: Equipment Alignment

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
SOP-0035	Reactor Core Isolation Cooling System	35
SOP-0049	120 VDC System	27
SOP-0053	Standby Diesel Generator and Auxiliaries	307
SOP-0058	Control Building HVAC Systems	20

TECHNICAL SPECIFICATIONS

<u>NUMBER</u>	<u>TITLE</u>	<u>AMENDMENT</u>
Technical Specification 3.7.3	Control Room Air Conditioning (AC) System	132
Technical Specification 3.8.1	AC Sources-Operating	156

Section 1R05: Fire Protection

CONDITION REPORTS

CR-RBS-2009-06002 CR-RBS-2009-06003

MISCELLANEOUS

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
	Pre-Fire Plan/Strategy Book	
DRL-FP-00107	Control Building 116-foot C71 P002 RPS DIST PNL	4
TRM 3.7.9.6	Fire-Rated Assemblies	5

USAR 9A.2	Fire Hazards Analysis	10
USAR 9.4.7.2.1	Fire Pump House Heating and Ventilation System	15

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
AOP-0009	Loss of Normal Service Water	16
EN-DC-127	Control of Hot Work and Ignition Sources	7
EN-DC-128	Fire Protection Impact Reviews	4
FPP-0100	Fire Protection System Impairment	10
FPP-0101	Fire Suppression System Inspection	10
SOP-0037	Fire Protection Water System Operating Procedure (System 251)	27

Section 1R07: Heat Sink Performance

CONDITION REPORTS

CR-RBS-2008-02821 CR-RBS-2008-02878 CR-RBS-2009-04345 CR-RBS-2009-04830

PROCEDURE

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
SOP-0042	Standby Service Water	28

UPDATED SAFETY ANALYSIS REPORT DOCUMENTS

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION/DATE</u>
USAR 1.2.2.8.1	Standby Service Water System	19
USAR 3.8.4.4.4	Standby Service Water Cooling Tower and Pumphouse	August 1987
USAR 7.3.1.1.8	Standby Service Water (SSW) System	13
USAR 9.2.7	Standby Service Water System	12

USAR Table 9.2-5	Decay Heat Rejection and Containment Unit Cooler Heat Load to Standby Service Water Following a Small Line Break Event SBA w/ Bypass	14
USAR Table 9.2-6	Plant Auxiliaries Heat Load (BTU/hr) Input to Standby Service Water Following a DBA	17
USAR Table 9.2-7	Plant Auxiliaries Heat Input to Standby Service Water	17
USAR Table 9.2-10	Total Integrated Heat Input to Standby Service Water from RHR Heat Exchangers, Containment Unit Cooler, Pumps, and Plant Auxiliaries	17
USAR Table 9.2-11	Standby Cooling Tower Performance Following a Large Recirculation Line Break	17
USAR Table 9.2-13	Standby Service Water System Safeguard Equipment Status Following a DBA	9
USAR Table 9.2-15	Standby Service Water System Major Component Design Data	14
USAR Table 9.2-16	Standby Service Water System Single Passive Failure Criterion Analysis	8A

Section 1R08: Inservice Inspection Activities

CONDITION REPORTS

CR-RBS-2008-00430	CR-RBS-2008-00511	CR-RBS-2009-04463	CR-RBS-2009-04613
CR-RBS-2009-04683	CR-RBS-2009-04770	CR-RBS-2009-04773	CR-RBS-2009-04850

MISCELLANEOUS

<u>NUMBER</u>	<u>TITLE</u>	<u>DATE</u>
ELO-LO-2006-00094	Welding Program Assessment	July 24, 2006

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
CEP-NDE-0400	Ultrasonic Examination	3

CEP-NDE-0404	Manual Ultrasonic Examination of Ferritic Piping Welds	4
CEP-NDE-0641	Liquid Penetrant Examination (PT) for ASME Section XI	5
CEP-NDE-0731	Magnetic Particle Examination (MT) for ASME Section XI	3
CEP-NDE-0903	VT-3 Examination	5

Section 1R12: Maintenance Effectiveness

CONDITION REPORTS

CR-RBS-2007-5298	CR-RBS-2008-3091	CR-RBS-2008-3303	CR-RBS-2008-3601
CR-RBS-2008-5073	CR-RBS-2008-5097	CR-RBS-2008-5129	CR-RBS-2008-5130
CR-RBS-2008-6244	CR-RBS-2008-6676	CR-RBS-2008-7029	CR-RBS-2008-7033
CR-RBS-2008-7039	CR-RBS-2008-7040	CR-RBS-2009-0424	CR-RBS-2009-0595
CR-RBS-2009-1485	CR-RBS-2009-5016	CR-RBS-2009-5048	CR-RBS-2009-5133
CR-RBS-2009-5170	CR-RBS-2009-5173	CR-RBS-2009-5314	CR-RBS-2009-5429
CR-RBS-2009-5474	CR-RBS-2009-6338	CR-RBS-2009-6348	

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
EN-DC-205	Maintenance Rule Monitoring	2
EN-DC-206	Maintenance Rule (a)(1) Process	1

Section 1R13: Maintenance Risk Assessment and Emergent Work Controls

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
ADM-0096	Risk Management Program and Implementation Risk Assessment	304
AOP-0029	Severe Weather Operation	24

AOP-0064	Degraded Grid	2
EN-MA-125	Troubleshooting and Control of Maintenance Activities	5
EN-OP-103	Reactivity Management Program	3
EN-WM-101	On-Line Work Management Process	6

Section 1R15: Operability Evaluations

CONDITION REPORTS

CR-RBS-2002-01095 CR-RBS-2008-02504 CR-RBS-2009-04959 CR-RBS-2009-05882
CR-RBS-2009-05928

MISCELLANEOUS

<u>NUMBER</u>	<u>TITLE</u>	<u>DATE</u>
IEEE Standard C37.91-2008	IEEE Guide for Protecting Power Transformers	May 30, 2008

PROCEDURE

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
STP-251-3205	Diesel Fire Pump Operational Test	14

Section 1R18: Plant Modifications

CONDITION REPORT

CR-RBS-2009-06517

MISCELLANEOUS

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
ARP-SBO-DIESEL	SBO Diesel Alarm Response	0 and 1
EC 18153	Replace Old Station Blackout Diesel, BYS-EG1, with New Generac Diesel Generator	0

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
EN-DC-115	Engineering Change Process	8
EN-DC-136	Temporary Modifications	4
EN-LI-100	Process Applicability Determination	7
OSP-0007	Preparation, Review and Revision of Operations Section Procedures	19
RBNP-001	Development and Control of RBS Procedures	29
SOP-0054	Contingency Equipment Operation	306 and 307

UPDATED SAFETY ANALYSIS REPORT DOCUMENTS

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION/DATE</u>
USAR 8.1.4	Onsite AC Systems	6
USAR 15.2.6.1.1.1	Loss of Normal and Preferred Station Service Transformers	August 1987

WORK ORDERS

<u>NUMBER</u>	<u>TITLE</u>
WO 00210006	Task 1, Install TMOD 6321 on RWCU A
WO 00210006	Task 3, Install TMOD 6321 on RWCU B
WO 00209226	Task 3, RTX-XSR1E, Remove Load Tap Changer and Install TMOD 17184
WO 00199598	Task 3 & 4, RTX-XSR1F Transformer Trouble Alarm Received in Main Control Room

Section 1R19: Postmaintenance Testing

MISCELLANEOUS

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
EC 15860	Disable Alarm and Trip Functions for Sudden Pressure Relays for RTX-XSR1F and RTX-XSR1D	0

UPDATED SAFETY ANALYSIS REPORT DOCUMENTS

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION/DATE</u>
USAR 8.1.4	Onsite AC Systems	6
USAR 15.2.6.1.1.1	Loss of Normal and Preferred Station Service Transformers	August 1987

WORK ORDERS

<u>NUMBER</u>	<u>TITLE</u>
WO 00209226	Task 7, EMRTX-XSR1E Post Installation Testing for TMOD 17814
WO 00199598	Task 3 & 4, RTX-XSR1F Transformer Trouble Alarm Received in Main Control Room

Section 1R20: Refueling and Other Outage Activities

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
AOP-0001	Reactor Scram	24
GMP-0102	Reactor Vessel Disassembly	18
GMP-0103	Reactor Vessel Assembly	20
GOP-0001	Plant Startup	20
SOP-0031	Residual Heat Removal	306

Section 1R22: Surveillance Testing

CONDITION REPORTS

CR-RBS-1986-00018 CR-RBS-2009-04681 CR-RBS-2009-06460

MISCELLANEOUS

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
ARP-SBO-DIESEL	SBO Diesel Alarm Response	1
ASME OMb Code-2006	Code for Operation and Maintenance of Nuclear Power Plants	

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
AOP-0027	Fuel Handling Mishaps	23
GMP-0108	Reactor Vessel Disassembly	31
SOP-0054	Contingency Equipment Operation	306
STP-309-0203	Division III Diesel Generator Operability Test	306
STP-309-0601	Division I ECCS Test	31

TECHNICAL SPECIFICATIONS

<u>NUMBER</u>	<u>TITLE</u>	<u>AMENDMENT</u>
Technical Specifications 3.8.1	AC Sources-Operating	156
Technical Specifications 3.8.2	AC Sources-Shutdown	132
Technical Specifications 3.8.3	Diesel Fuel Oil, Lube Oil, and Starting Air	81

WORK ORDERS

<u>NUMBER</u>	<u>TITLE</u>
WO 52037115	Miscellaneous Containment Isolation Valves Quarterly Operability Test
WO 52228449	Diesel Fire Pump Battery Weekly Surveillance Test

Section 20S2: ALARA Planning and Controls

AUDITS, SELF-ASSESSMENTS, AND SURVEILLANCES

<u>NUMBER</u>	<u>TITLE</u>	<u>DATE</u>
LO-RLO-2009-00005	Focused Assessment Report; Occupational Radiation Safety	March 06, 2009
LO-RLO-2009-00057	Focused Assessment Report; Occupational Radiation Safety	August 07, 2009

<u>NUMBER</u>	<u>TITLE</u>	<u>DATE</u>
Snapshot Assessment	RP Outage Readiness	May 26, 2009
Snapshot Assessment	Dose Reduction Techniques	August 31, 2009
Snapshot Assessment	Control and Use of Contaminated Drip Bags	May 12, 2009
Snapshot Assessment	Access to High Radiological Risk Activities	September 30, 2009

CONDITION REPORTS

CR-RBS-2009-00309	CR-RBS-2009-01843	CR-RBS-2009-02325	CR-RBS-2009-02414
CR-RBS-2009-02854	CR-RBS-2009-03155	CR-RBS-2009-03918	CR-RBS-2009-04440
CR-RBS-2009-04779	CR-RBS-2009-04817	CR-RBS-2009-04948	CR-RBS-2009-05043
CR-RBS-2009-05064	CR-RBS-2009-05169	CR-RBS-2009-05749	CR-RBS-2009-05751
CR-RBS-2009-05844			

MISCELLANEOUS

<u>NUMBER</u>	<u>TITLE</u>
2009-2013	River Bend Station CRE Goals

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
EN-RP-100	Radworker Expectations	3
EN-RP-101	Access Control for Radiologically Controlled Areas	4
EN-RP-102	Radiological Control	2
EN-RP-105	Radiation Work Permits	7
EN-RP-109	Hot Spot Program	3
EN-RP-141	Job Coverage	4

RADIATION WORK PERMITS

<u>NUMBER</u>	<u>TITLE</u>
2009-1006	General Radwaste Activities including shipping <1000 mrem
2009-1214	General Radwaste Activities including shipping and LHRA
2009-1432	RF-15 Snubber Work
2009-1603	RF-15 Noble Chemistry Project Activities
2009-1800	RF-15 Refuel Activities
2009-1938	RF-15 Scaffold and Shielding
2009-1943	RF-15 Drywell

Section 2PS2: Radioactive Material Processing and Transportation

AUDITS, SELF-ASSESSMENTS, AND SURVEILLANCES

<u>NUMBER</u>	<u>TITLE</u>	<u>DATE</u>
QA-15-2007-RBS-1 & -2	Quality Assurance Audit of Radwaste and Radiation Protection Programs	January 3, 2008
QA-15-2007-RBS-1	Followup to Audits	July 14-17, 2008
QS-2008-RBS-11	Followup of the 2008 Radwaste Surveillance	December 15-17, 2008

CONDITION REPORTS

CR-HQN-2007-0541	CR-HQN-2007-0939	CR-HQN-2007-0939	CR-HQN-2007-0978
CR-HQN-2008-0013	CR-HQN-2008-0023	CR-HQN-2009-0400	CR-RBS-2007-3140
CR-RBS-2007-3661	CR-RBS-2007-3776	CR-RBS-2007-3793	CR-RBS-2007-4679
CR-RBS-2008-0452	CR-RBS-2008-3446		

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
EN-RW-102	Radioactive Shipping Procedure	6

EN-RW-104	Scaling Factors	5
EN-RW-105	Process Control Program	1
RWS-0304	Radioactive Waste Handling and Control	15

RADIOACTIVE MATERIAL SHIPMENTS

2008-039	2009-040	2009-105	2009-106	2009-107
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10 CFR Part 61 Waste Stream Scaling Factors

<u>TITLE</u>	<u>DATE</u>
Spent Pool Fuel Cleanup (SFC 2009)	June 9, 2009
Westinghouse Chemical Decon	March 1, 2008
Dry Active Waste	August 7, 2008
Condensate and Liquid Waste System	January 21, 2009

Section 40A1: Performance Indicator Verification

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
EN-LI-102	Corrective Action Process	13
EN-LI-114	Performance Indicator Process	4

Section 40A2: Identification and Resolution of Problems

CONDITION REPORTS

CR-RBS-2003-00670	CR-RBS-2008-02182	CR-RBS-2008-05650	CR-RBS-2009-04175
CR-RBS-2009-05933			

LICENSEE EVENT REPORTS

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
LER 1995-012	Manual Scram Due to Recirculation Pump Transient	0
LER 2009-002	Unplanned Manual Reactor Scram Following Trip of Both Reactor Recirculation Pumps	0

MISCELLANEOUS

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
CEO 2009-00005	Nuclear Oversight Fleet Quarterly Report	0
ER-RB-2002-0042	Evaluation of Recirculation Pump Downshift Logic	0

<u>TITLE</u>	<u>DATE</u>
Caution Tags Greater than 90 day Index	August 31, 2009 October 31, 2009
Clearances Greater than 90 day Index	August 31, 2009 October 31, 2009
Control Room Annunciator Index	August 31, 2009 October 31, 2009
Control Room Deficiency Index	August 31, 2009 October 31, 2009
Operations Aggregate Index	August 31, 2009 October 31, 2009
Operator Burden Index	August 31, 2009 October 31, 2009
Operator Work Around Index	August 31, 2009 October 31, 2009
ODMI Index	November 6, 2009
ODMI, FWS P1A/B/C Operation with High Seal Leakage	
ODMI, High Alterx Housing Vibration	
ODMI, Turbine # 2 Valve Packing Leak	
Temporary Modification Log	

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
AOP-0024	Thermal Hydraulic Stability Controls	22
GOP-0004	Single Loop Operation	20

Section 40A3: Event Follow-Up

CONDITION REPORTS

CR-RBS-2003-00670 CR-RBS-2008-02182 CR-RBS-2008-05650 CR-RBS-2009-04175
CR-RBS-2009-05933

MISCELLANEOUS

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
ER-RB-2002-0042	Evaluation of Recirculation Pump Downshift Logic	0
LER 1995-012	Manual Scram Due to Recirculation Pump Transient	0
LER 2009-002	Unplanned Manual Reactor Scram Following Trip of Both Reactor Recirculation Pumps	0

PROCEDURES

<u>NUMBER</u>	<u>TITLE</u>	<u>REVISION</u>
AOP-0024	Thermal Hydraulic Stability Controls	22
GOP-0004	Single Loop Operation	20