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 JRBuchanan
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NOVEMBER 29 1979

Docket No. 50-3

Mr. William J. Cahill, Jr.
 Vice President
 Consolidated Edison Company
 of New York, Inc.
 4 Irving Place
 New York, New York 10003

Dear Mr. Cahill:

In a recent discussion with you, we discussed certain aspects regarding the location of Indian Point Station Unit No. 2 in areas of relatively high population density. Specifically, we indicated that the staff was considering an in-depth review of that facility which would extend beyond the review currently in progress of your emergency preparedness capabilities. Various alternatives would be considered that would be effective in reducing the probability and consequences of a severe reactor accident. The NRC review will also include the Indian Point Station Unit No. 3 and Zion Nuclear Station Unit Nos. 1 and 2 facilities. As a first step in this activity, an initial joint meeting with the licensees of all of these facilities is scheduled for December 5, 1979, at 10:30 a.m. at our Bethesda office. We understand that Mr. Cordell Reed of Commonwealth Edison Company will be coordinating this meeting for all three licensees.

While we will provide the details of our intended review and will discuss your role at the upcoming meeting, prior to the meeting, it would be helpful for you to give consideration to the various inherent design features in these plants that would be effective for the mitigation of a severe reactor accident. To further assist you in preparation for the meeting, attached is a listing of considerations that the staff will be exploring.

Sincerely,

151

Darrell G. Eisenhut, Acting Director
 Division of Operating Reactors

Enclosure:
 As stated

cc w/encl:
 See next page

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OFFICE	DOR <i>B</i>	DOR <i>D</i>	<i>Doc: 02871</i>		
SURNAME	GGZech: l b	D Eisenhut	L. Olschan		
DATE	11/28/79	11/29/79	11/28/79		



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

November 29, 1979

Docket No. 50-3

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Vice President
Consolidated Edison Company
of New York, Inc.
4 Irving Place
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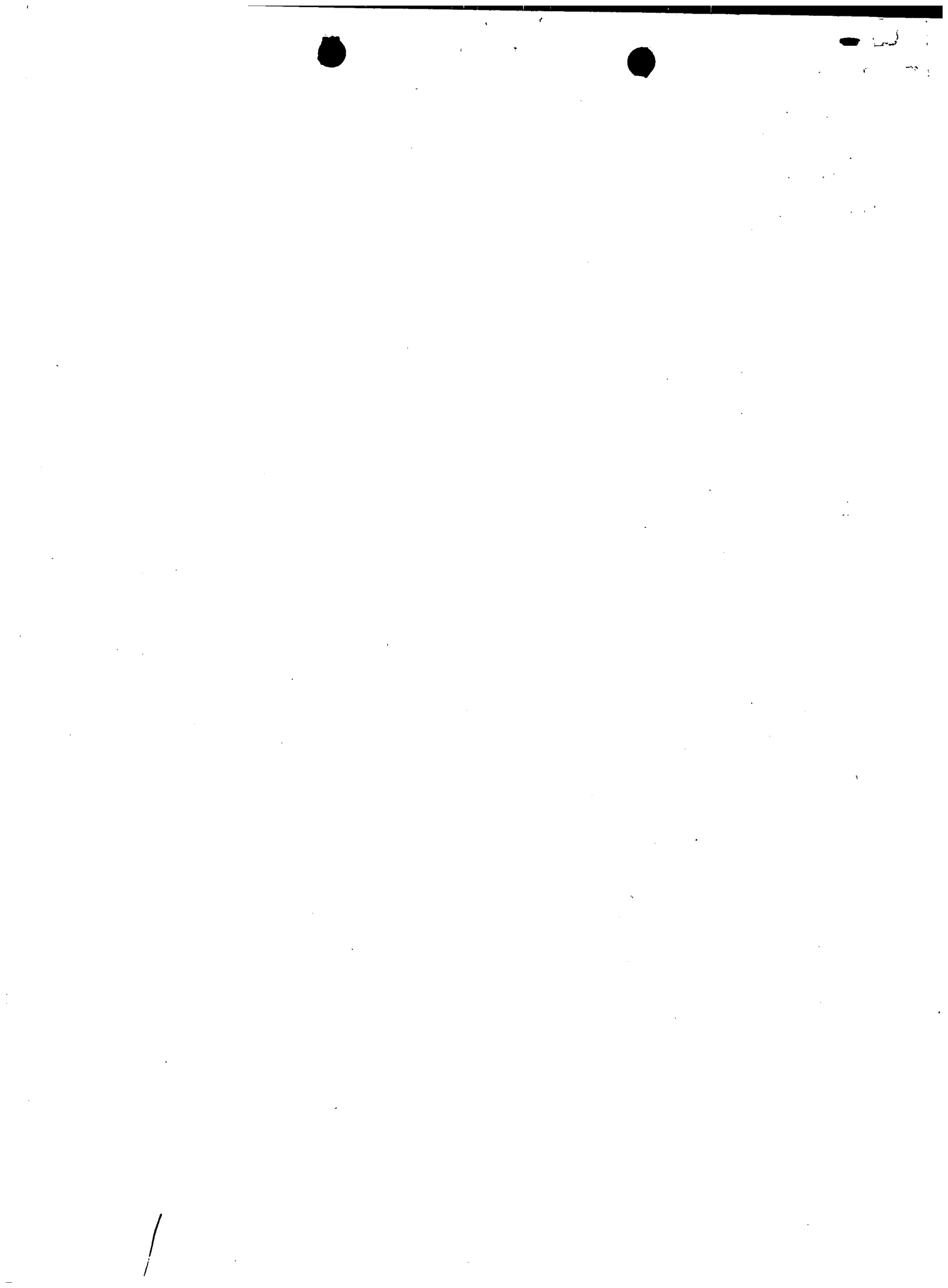
Sincerely,

A handwritten signature in cursive script that reads "Darrell G. Eisenhower".

Darrell G. Eisenhower, Acting Director
Division of Operating Reactors

Enclosure:
As stated

cc w/encl:
See next page



Mr. William J. Cahill, Jr.
Consolidated Edison Company of New York, Inc.

- 2 -

November 29, 1979

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Office of Radiation Programs (AW-459)
U. S. Environmental Protection Agency
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U. S. Environmental Protection Agency
Region II Office
ATTN: EIS COORDINATOR
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New York, New York 10007

STAFF CONSIDERATIONS

FOR DECEMBER 5, 1979 MEETING

1. Improved Interim Operational Actions: These actions involve, but are not necessarily limited to, such considerations as increased IE inspections, additional resident inspectors, augmented control room staffing and improved operator qualifications and training. Selected aspects from the Long Term Lessons Learned Report (NUREG-0585) may also be included as measures to further reduce the likelihood of a severe accident, e.g., corporate management accountability. In addition, a review of the emergency procedures would be included to improve operational capabilities. This effort will also include an examination of the derating of the plant during an interim period while the actions in items (2) and (3) below are reviewed.
2. Current Licensing Actions: The actions involved in this aspect of the plan will include implementation on a priority basis of those TMI-2 short term lessons learned actions stated in NUREG-0578 and Bulletin and Order review matters. In addition prompt attention will be given to processing of outstanding amendments for these plants. Subsequent actions would include implementation of any recommendations resulting from the resolution of unresolved safety items on a priority basis.
3. Severe Accident Mitigation Features: In addition to the increased preventive actions described in (1) and (2) above, there are several measures that may be effective in mitigating the consequences of a major reactor accident involving core melt. Additional means such as derating and measures to augment the existing ECCS via an independent system are considerations that fall between prevention and mitigation and may be effective agents against a major nuclear accident. Such means also would include a bunkered system with a dedicated heat removal system.