1997 ANNUAL REPORT

MMP 91-3-172 FP-FRW, REV. 1

MAIN TURBINE GENERATOR
FIRE PROTECTION SYSTEM UPGRADE

Description and Purpose

This modification converted the manually operated water sprinkler system to an automatic mode of operation and the automatic CO2 system to a manually operated backup system for the Governor Housing and Turbine Bearings.

Summary of Safety Evaluation

This modification was performed in accordance with IP3 Quality Assurance Program, made no changes to the Security System, accident analysis, safety analysis or the Technical Specifications. This modification improves the fire protection System of the Main Turbine Generator Bearings. It was concluded that this modification does not involve an unreviewed safety question.

1997 ANNUAL REPORT

NSE 92-3-264 RCS, REV. 1

CORE RELOAD FOR CYCLE 10
BELOW THE ELECTRICAL TUNNEL

Description and Purpose

This NSE addressed plant operation up through core reload to confirm that the new fuel could be loaded into the reactor vessel in the loading pattern designed for the Cycle 10 refueling outage.

Summary of Safety Evaluation

The Cycle 10 core is generally the same in design and function as all previous cores and does not create any new scenarios. This NSE evaluates each of the three phases of the core reload; Fuel Selction, Core and Cycle Design; Fuel Design and Fabrication; and Core Reload, and deonstrates that none of them adversely affects the safety of the plant or involves an unreviewed safety question.

1997 ANNUAL REPORT

NSE 93-3-205 RM, REV. 2

IP3 RADIATION MONITORING SYSTEM UPGRADE

Description and Purpose

This NSE replaced R-1 (Control Room (CR) Particulate Radiation Monitoring) and R-33 (CR Noble Gas Radiation Monitoring) with upgraded monitors and retired R-32 (CR Radioactive Particulate Monitoring). These changes were made as a result of NRC Violation 91-22-01, which resulted in the identification of the need for upgrades or replacement due to a high failure rate of those monitors.

Summary of Safety Evaluation

The replacement of R-1 and R-33 improved monitoring reliability of control room radiation and satisfied the identified NRC violation. Retirement of R-32 was acceptable because monitoring of particulate levels in the control room is not required.

1997 ANNUAL REPORT

MMP 94-3-126 HD, REV. 0

REPLACEMENT OF DAMAGED QA CATEGORY I CABLES IN SERVICE WATER PUMP FEEDER DUCTBANK

Description and Purpose

The purpose of this modification was to replace two ASCO solenoid operated valves, SOV-1404 and SOV-1405, with the proper model to be consistent with the original design.

Summary of Safety Evaluation

The subject solenoid operated valves (SOV) control the instrument air supply to FCV-1204 and 1205 which regulate quench water flow to the heater drain pumps. The installed SOVs were not consistent with the original design which caused the inability of FCV-1204 and 1205 to meet system requirements. The new SOVs 1404 and 1405 are for "universal" type operation which is consistent with the original design and provide the necessary throttle control required for this application.

1997 ANNUAL REPORT

NSE 94-3-191, REV. 2

INDIAN POINT 3 AND WHITE PLAINS OFFICE - QUALITY ASSURANCE (QA) REORGANIZATION

Description and Purpose

This evaluation ensures that no unreviewed safety question exists in re-organizing the Indian Point 3 and White Plains QA departments. These changes involve title changes, the creation and deletion of positions, and the reassignment of position responsibilities and reporting relationships.

Summary of Safety Evaluation

These changes are administrative in nature and do not involve plant equipment or operating conditions.

1997 ANNUAL REPORT

MMP 94-3-250 PZR, REV. 0

REPLACE PRESSURIZER RELIEF TANK LEVEL TRANSMITTER

Description and Purpose

This modification replaced a displacer type level TRANSMITTER, installed on the Pressurizer Relief Tank, with a delta-P type level TRANSMITTER. The delta-P type was used because the original model was inoperable and obsolete.

Summary of Safety Evaluation

The new TRANSMITTER is more accurate and provides the same signals and functions as the original model. The TRANSMITTER does not provide safety significant functions, therefore, its replacement has no unreviewed safety questions.

1997 ANNUAL REPORT

NSE 95-3-237, REV. 0

CONTROL ROOM UPGRADE

Description and Purpose

This safety evaluation ensured that upgrades (and installation) to the Control Room workstations, Page Party System and telephones had no safety significance. This work was done by MMP 95-3-237.

Summary of Safety Evaluation

The changes made by this minor mod were improvements and did not have any safety significance. Due to proper scheduling and coordination, interruption of daily operations of the Control Room were avoided.

1997 ANNUAL REPORT

NSE 95-3-241 SWS, REV. 1

CONTROL ROOM AIR CONDITIONING SERVICE WATER PIPING UPGRADE

Description and Purpose

The purpose of this evaluation was to evaluate and provide justification for the replacement of portions of existing Service Water piping supplying the Control Room Air Conditioning (CRAC) units.

Summary of Safety Evaluation

The modification replaced all of the cement lined carbon steel sections of Service Water Lines 1223, 1224, and 1231 from lines 1093 and 1094 up to and including valve SWN-95. The replacement pipe material was austenitic stainless steel with 6% molybdenum (Mo), a corrosion resistant material. The modification does not change the overall design of the Service Water supply to the CRAC Unit Condensers. The change in material of the piping and valves does not impact the design or operation of the system. The change improves system reliability.

1997 ANNUAL REPORT

NSE 95-3-242 HD, REV. 1

HEATER DRAIN PUMP SUCTION STRAINERS

Description and Purpose

This minor modification installed permanent wire basket-type strainers in the heater drain pumps (HDP) 31 and 32 suction piping to protect the pumps from foreign material and debris which may be present from maintenance activities.

Summary of Safety Evaluation

The heater drain pumps are not safety significant nor are the strainers.

The strainers are an improvement that provides additional protection to the heater drain pumps from maintenance debris.

1997 ANNUAL REPORT

MMP 95-3-250, REV. 0

COMPONENT COOLING WATER PUMPS VENT VALVE ARRANGEMENT CHANGE

Description and Purpose

The purpose of this modification was to improve the reliability and overall operability of the three Component Cooling Water pumps by upgrading their casing vent valves and mechanical seals flush line configuration. A common mode of pump maintenance has been replacement of the mechanical seals. The scope of this modification involved the utilization of the upper casing vent connection on each CCW pump to supply seal/flush water to the mechanical seals, as well as perform its venting function.

Summary of Safety Evaluation

The Safety Evaluation demonstrated that the modification to the CCW System does not affect overall plant safety. In addition, this modification complies with the overall system design criteria contained in the FSAR and does not change the Plant Technical Specifications. This modification improves the reliability and operability of the three CCW pumps by upgrading their casing vent valves and mechanical seals flush lines configuration.

1997 ANNUAL REPORT

MMP 95-3-259, REV. 0

#36 EXTRACTION STEAM LINE REPLACEMENT

Description and Purpose

The purpose of this modification was to replace the extraction steam carbon steel piping to the No.36 Feedwater Heaters (FWH's)in order to eliminate concerns of pipe wall thinning due to Flow Accelerated Corrosion (FAC). The carbon steel pipe was replaced with carbon steel pipe clad with stainless steel cladding, which enhances the reaction to erosion/corrosion in this flow regime.

Summary of Safety Evaluation

This modification eliminates concerns of pipe wall thinning due to Flow Accelerated Corrosion (FAC) in the No. 36 Extraction Steam Line. The replacement piping has the same schedule, size, configuration and routing as the existing piping. As a result of this modification, FAC will be practically eliminated in the No. 36 Extraction Steam piping. This modification resulted in a change in a flow diagram as identified in the FSAR. This modification does not have any other impact on the FSAR. Therefore, as a result of this review, it was determined that this modification does not pose an unreviewed safety question.

1997 ANNUAL REPORT

NSE 95-3-273 RCS, REV. 0

IN-VESSEL AND EX-VESSEL DOSIMETRY

Description and Purpose

This NSE evaluated the installation of two temporary dosimeters in the reactor vessel's existing guide basket and 4 sapphire neutron detectors in the annular space between the reactor vessel and the containment building. They will be in place for the duration of Fuel Cycle 10 and will be removed for evaluation during Refueling Outage 10. These dosimeters are temporary and will not be returned once they are removed.

Summary of Safety Evaluation

Both in-vessel and ex-vessel dosimeters and detectors are structurally secured and do not interfere with any safety related system, structure or components.

1997 ANNUAL REPORT

NSE 95-3-287, REV. 1

EMERGENCY DIESEL GENERATOR FAN MOTOR REPLACEMENT AND POWER REALIGNMENT

Description and Purpose

The purpose of this modification was to reconfigure power feeds to the Emergency Diesel Generator (EDG) exhaust fans, install additional Motor Control Centers (MCC) and upgrade the EDG exhaust fan motors. This modification was installed to permanently correct known deficiencies with the existing design.

Summary of Safety Evaluation

The changes implemented in this modification will permanently make the power supplies to the EDG exhaust fans independent of each other. Additionally, EDG Fan Motors are properly sized to carry their design loads, so that they will operate at lower temperatures, which will increase the motor life and associated reliability. The changes will not significantly affect the total load on the electrical distribution system but simply change the way the existing load is distributed among the Class 1E components.

1997 ANNUAL REPORT

NSE 95-3-303 SWS, REV. 1

FCU MOTOR COOLER RETURN PIPING REPLACEMENT

Description and Purpose

The purpose of this evaluation was to evaluate modification MMP 95-3-303 SWS which replaced the Fan Cooler Unit (FCU) Motor Cooler return piping, service water lines nos. 495 through 499 from the flanges at the downstream side of the five FCU Motor Cooler discharge valves to the flanges downstream of the five SWN 71-1 through -5 containment isolation valves.

Summary of Safety Evaluation

The service water lines nos. 495 through 499 were replaced from the flanges at the downstream side of the five motor cooler discharge valves (SWN-521, -523, -525, -527, -529) to the flanges downstream of the five SWN 71-1 through -5 containment isolation valves. The internals of containment penetration "SS" were removed and replaced, and local temperature indicators TI-1309 through -1313 were replaced. The piping, valves, and temperature indicators were replaced with austenitic stainless steel with 6% molybdenum (Mo), a more corrosion resistant material than the original cement-lined carbon steel. The replacement piping and valves provide the same function as the original relative to function, pressure class, nominal diameter, and pressure retaining capability.

1997 ANNUAL REPORT

NSE 95-3-304 SI, REV. 0 & 1

EVALUATION OF CHANGES TO EMERGENCY OPERATING PROCEDURES (EOPS) MADE DURING THE RCIP/RS94/R09 OUTAGES

Description and Purpose

Rev. 0 and 1 of this NSE justified the changes made to the EOPS during the RCIP/RS94/R09 outages. Those changes included various setpoint changes, modifications made to the plant during the above outages, step sequence changes, procedure enhancements, and editorial changes.

Summary of Safety Evaluation

The EOP changes described in this NSE were verified and validated on the Indian Point 3 simulator. The simulator showed that the EOP improvements effectively brought the plant to a safe stable condition.

1997 ANNUAL REPORT

NSE 95-3-325 GWD, REV. 1

WD-PCV-1069D REPLACEMENT AND ADDITION OF BYPASS VALVE WD-1077

Description and Purpose

This minor modification replaced the Reactor Coolant Drain Tank (RCDT) pressure regulator. While attempting to sample the RCDT, it was discovered that sample flow was not reaching the Waste Gas analyzer. To facilitate sampling, pressure regulator WD-PCV-1069D was removed and had been replaced by 3/8" stainless tubing under Temporary Modification 95-03747-01. A new Veriflow HFR 900K series pressure regulator has been installed, and the TM has since been removed. A bypass line around the pressure regulator has also been provided to facilitate gas sampling in the case of pressure regulator malfunction.

Summary of Safety Evaluation

This mod restores the RCDT gas sampling to meet its original design parameters. In addition, system operability is improved by the addition of the bypass line by providing a back-up means of sampling.

1997 ANNUAL REPORT

NSE 96-3-033 SWS, REV. 3

POWER OPERATION WITH
SERVICE WATER TEMPERATURE <35°F

Description and Purpose

Revisions 1, 2 & 3 of this NSE identified the concerns associated with operating the plant with Service Water temperature at 28°F and was included in the 1996 Annual Report. Revision 3 to this NSE included calculations ensuring that colder river temperatures had no detrimental effects on SWS fluids such as lube oil and cooling water.

Summary of Safety Evaluation

The calculations concluded that lube oil temperatures will remain above its pour point temperature of 10°F, and that the cooling water within the SWS piping will continue to flow with a river water temperature of 28°F.

1997 ANNUAL REPORT

NSE 96-3-060 CW, REV. 1

RETIREMENT OF OUTLET WATERBOXES LEVEL CONTROLLERS AND ASSOCIATED CONDENSER OUTLET WATERBOX LOW LEVEL ALARM

Description and Purpose

This minor modification retired the level controllers located on the outlet waterboxes of the main condensers and its associated low waterbox level alarm in the Control Room. Retirement of the level controllers eliminates a nuisance alarm in the Control Room.

Summary of Safety Evaluation

Operation of the waterboxes are not included in the Safety Analysis Report therefore, the retirement of the level controllers are not safety significant. If waterbox level is desired, the inlet waterbox level and circulating pump motor current can be used to determine level.

1997 ANNUAL REPORT

NSE 96-3-064 ADMIN, REV. 5

REORGANIZATION OF MANAGEMENT STRUCTURE WITHIN NUCLEAR GENERATION

Description and Purpose

This evaluation ensures that no unreviewed safety question exists in re-organizing the New York Power Authority Management structure within Nuclear Generation. These changes involve title changes, the creation and deletion of positions, and the reassignment of position responsibilities and reporting relationships.

Summary of Safety Evaluation

These changes are administrative in nature and do not involve plant equipment or operating conditions.

1997 ANNUAL REPORT

NSE 96-3-065, REV. 1

ELECTRICAL TUNNEL SPRAY SYSTEM IMPROVEMENT

Description and Purpose

The changes to the electrical Tunnel preaction water spray systems were necessary to improve system design and performance to the extent necessary to conform to the provisions of Branch Technical Position (BTP) APCSB 9.5-1, Appendix A and the system design performance relied upon by the NRC staff in the granting of 10CFR50.12 specific exemptions from the requirements of Section III.G of Appendix R to 10CFR50. The further purpose of the Nuclear Safety Evaluation was to review the changes to the Appendix R Safe Shutdown Analysis and to plant operating procedures affected by the changes in the Analysis. Revision 1 of the NSE reviewed the performance of a hydrostatic pressure test of each system's piping and fittings on plant safety.

Summary of Safety Evaluation

The Changes imposed by this modification involved an upgrade to the design and performance of the existing fire protection water spray systems serving the 33' and 43' elevations of the Electrical Tunnel. Additionally, the Appendix R safe shutdown analysis was revised to demonstrate assurance of safe shutdown capability independent of the electrical tunnel entryway. The revision to the analysis basically combines two analysis areas in the Electrical Tunnel entryway into one. As a result of this change, revisions to operating procedures and operational specifications were required. The focus of the change was to ensure compliance with the requirements of Appendix R to 10CFR50.

1997 ANNUAL REPORT

MMP 96-3-119 PS, REV. 0

REPLACEMENT OF PRIMARY SAMPLING SYSTEM
VALVE SP-PCV-1300C

Description and Purpose

The purpose of this evaluation was to evaluate the replacement of Primary Sampling System flow control valve SP-PCV-1300C and to demonstrate that the valve replacement did not represent a change to the plant Technical Specifications

Summary of Safety Evaluation

Minor Modification 96-3-119PS replaced the existing Primary Sampling flow control valve PS-PCV-1300C. The original valve was a 3/8" needle valve and was replaced with a 1/4" needle valve. The original valve had a history of repeated failures which prevented proper flow control during Reactor Coolant System sampling. It was determined that the valve was too large for the service which required excessive throttling resulting in seat and plug damage causing flow control and isolation problems. The tubing design was evaluated using Engineering Standard CES-1 and it was determined that the design would not be affected by the valve replacement.

1997 ANNUAL REPORT

NSE 96-3-139 ED 480V, REV. 0

APPENDIX R DIESEL GENERATOR (D/G) 480V AUXILIARIES UPGRADE CONDENSATE PUMPS

Description and Purpose

This NSE replaced Main Circuit Breaker (MCB) #2C4 (for Appendix R D/G Auxiliaries support systems) with a fusible disconnect switch. This was done as a result of a race to trip condition between the feeder breaker (MCB #2C4) and the downstream circuit breakers (MCC #314) which had the potential to cause a loss of all the D/G auxiliaries and subsequent trip or unavailability of the Appendix R D/G.

Summary of Safety Evaluation

This modification improves reliability and availability of the Appendix R D/G to provide emergency backup power in the event of loss of offsite power and the failure of the Emergency Diesel Generators.

1997 ANNUAL REPORT

MMP 96-3-146 CCW, REV. 0

REPLACEMENT OF TEMPERATURE CONTROL VALVE AC-TCV-130

Description and Purpose

This minor modification replaced the 6" butterfly valve (Temperature Control Valve AC-TCV-130 valve) with a Neles-Jamesbury R-Series 6"-Ball Valve with Q-Trim™. The original valve was oversized for the intended throttling application and had caused cavitation and damage to the valve trim and downstream piping producing noise and high vibration.

Summary of Safety Evaluation

The replacement valve does not change the original design function and is more appropriate for this application. It provides better system functioning and reliability by preventing cavitation and reducing stress on the associated piping.

1997 ANNUAL REPORT

NSE 96-3-150 CVCS, REV. 0

EVALUATION FOR THE CONTINUED USE OF THE MODIFIED CONFIGURATION OF CONTAINMENT ISOLATION VALVES CH-MOV-250 A THROUGH D

Description and Purpose

This NSE supports continued use of the modified IVSW configuration and allows the modified IVSW to become the new licensing/design basis. The modified configuration was installed per MMP 93-3-442 CVCS, Rev. 1 and NSE 94-3-073 CVCS and was included in the 1995 Annual Report. NSE 94-3-073 CVCS was only valid until the end of the R09 refueling outage.

Summary of Safety Evaluation

Continued use of the modified IVSW configuration complies with the General Design Criteria for nuclear power plants as stated in Appendix A of 10 CFR 50 as well as ANS Standard 56.2. Thus, the modification is acceptable as the new licensing/design basis.

1997 ANNUAL REPORT

NSE 96-3-175 FP, REV. 1

REVISION TO AP-27.3 (RENUMBERED TO AP-64) AND FSAR SECTION 9.6.2.1

Description and Purpose

This NSE renumbered AP-27.3 (IP3 Site Fire Protection) to AP-64 and updated FSAR section 9.6.2.1 to reflect the new procedure number. Revision 0 of this NSE was reported in detail in the 1996 Annual Report.

Summary of Safety Evaluation

The changes made to this revision are editorial in nature and do not have any safety significance.

1997 ANNUAL REPORT

NSE 96-3-185 WCCPP, REV. 2

REPLACEMENT OF WCCPPS NITROGEN REGULATORS AND RELIEF VALVES

Description and Purpose

The purpose of this evaluation was to evaluate the replacement of the existing dome loaded nitrogen regulators, PS-PCV-1194, 1196, 1198, and 1200 with Tescom self venting pressure regulators. Also the replacement of nitrogen relief valves in response to concerns raised by Deviation Event Report (DER) 96-1559.

Summary of Safety Evaluation

The existing Cashco regulators and associated dome loading regulators were replaced with Tescom 44-1300 series regulators. The Tescom regulator is a high flow balanced regulator designed to minimize the effects of inlet pressure fluctuations on the outlet pressure. The regulator is a self-contained, direct-acting, spring-loaded and self-venting pressure regulator. The nitrogen relief valves downstream of the nitrogen regulators was replaced with Lonergan relief valve with a set pressure of 175 psig and a capacity of 1250 scfm. The regulator has an internal adjustable vent valve which will prevent pressure buildup in the downstream piping due to regulator seat leakage. The relief valve will protect the low pressure piping system downstream of the nitrogen regulator. The modified piping system utilized components suitable and compatible for the service and was evaluated for pressure, temperature, deadweight and seismic loads and complies with the requirements of ANSI B31.1.

1997 ANNUAL REPORT

NSE 96-3-204 RCS, REV. 0

PRESSURIZER SPRAY BYPASS VALVES RC-593 & RC-596 REPLACEMENT

Description and Purpose

The purpose of this evaluation was to evaluate the replacement of manually operated globe valves RC-593 and RC-596 with throttling valves for better flow control.

Summary of Safety Evaluation

The modification replaced the existing 3/4" globe valves in the inner bypass lines around each pressurizer spray control valve. The replacement valves provide a precise amount of flow relative to the number of turns of the handwheel. This eliminates the excess and uncertain flow which the existing globe valves permitted, thereby minimizing over usage of the heaters in the pressurizer. The outer bypass lines which tied in upstream and downstream of the isolation valves on both 3" legs of the spray line were also removed. The replacement of RC-593 and RC-596 provide a means to better control the flow rate. The removal of the outer bypass eliminates an additional flow path for potential leakage in the Reactor Coolant System (RCS).

1997 ANNUAL REPORT

NSE 96-3-220, REV. 0

REMOVAL OF INTERNALS, ISOLATION AND VENTING OF AIR TO DIAPHRAGM, AND CHANGING OF PACKING FOR VALVE SWN-PCV-1297

Description and Purpose

The purpose of this NSE was to evaluate the acceptability of removing the internals from SWN-PCV-1297, isolating and venting air to the valve actuator to fail the valve open changing the valve packing from Teflon to grafoil. SWN-PCV-1297 is one of the two control valves that limit service water pressure at the inlet of the CCR Air Conditioning Condensers (31 and 32).

Summary of Safety Evaluation

Removing the internals of valve SWN-PCV-1297 isolating and venting air to the valve diaphragm and changing the valve packing does not increase the consequences of an accident previously evaluated in the FSAR. This change does not adversely affect the ability of the CCR AC system to perform its intended function nor does it affect the ability of the Service Water System to perform its intended function during normal operation or post-accident conditions. Service water flow is maintained as required to all components.

1997 ANNUAL REPORT

NSE 96-3-223FP, REV. 1

CHANGE IN FIRE BRIGADE LEADER

Description and Purpose

This NSE changed the qualifications and assignment of the IP3 Fire Brigade leader. This revision requires the Fire Brigade leader to hold a current NRC Reactor Operator License and meet the minimum qualification requirements of a Fire Brigade Leader as defined in the Fire Protection Program. This revision also placed a limitation on the position such that, the Fire Brigade Leader will not concurrently fill a shift position to meet the minimum shift crew composition or a position assigned other essential functions during a fire emergency.

Summary of Safety Evaluation

The changes made to the Fire Brigade Leaders qualification requirements does not adversely affect the ability to respond to a plant fire or contribute to any accidents analyzed in the FSAR.

1997 ANNUAL REPORT

NSE 96-3-224 AFW, REV. 1 & 2

AUXILIARY FEEDWATER SYSTEM PURGE VOLUME

Description and Purpose

Rev. 0 of this NSE ensured that discrepancies between the analyzed value of the Auxiliary Feedwater System Purge volume and the purge volume, as calculated from piping drawings had no effect on plant operations (Rev. 0 was included in the 1996 Annual Report). Revisions 1 & 2 of this NSE ensured that the incorporation of plant specific calculations, performed by Westinghouse, which changed the maximum Condensate Storage Tank temperature from 100°F to 120°F, is an acceptable limit.

Summary of Safety Evaluation

The analytical changes made in Revision 1 & 2 more precisely quantify the effects of a loss of normal feed (LONF) event and establish criteria to maintain the LONF scenario as a Condition II event. As such, these analytical changes are not safety significant.

1997 ANNUAL REPORT

MOD 96-3-250 FHS, REV. 0

MANIPULATOR CRANE UPGRADE

Description and Purpose

The purpose of this evaluation was to ensure that the newly installed Manipulator Crane Controls Upgrade conformed with the existing safety analysis report requirements. The change upgraded the control systems on the manipulator crane to a more current industry-proven design.

Summary of Safety Evaluation

The design replaced the existing control system on the manipulator crane with industry proven reliable PLC based controls. The platform was also fitted with an IBM compatible computer allowing semi-automatic positioning of the manipulator crane bridge and trolley. The associated periphery control devices, such as limit switches, motor drives, etc., were replaced with reliable industry-proven qualified products.

1997 ANNUAL REPORT

NSE 96-3-259 SI, REV.0

MOTOR OPERATOR UPGRADE FOR SI-MOV-1802 A&B CONDENSER

Description and Purpose

This NSE evaluated the replacement of Limitorque actuators (type SMB-00 with a larger type SMB-0) on valves SI-MOV-1802A&B, to increase the capability of the Motor Operated Valves (MOV's) actuator. This will ensure that the valves will open at maximum design basis differential pressure. This upgrade was necessary to comply with Generic Letter 89-10 and NRC Information Notice 96-48.

Summary of Safety Evaluation

The valve upgrade improves the capability of the valves and does not change its designed function.

1997 ANNUAL REPORT

NSE 96-3-262 WCCPP, REV. 1

WELD CHANNEL AND CONTAINMENT PENETRATION REGULATOR REPLACEMENT

Description and Purpose

The purpose of this evaluation was to evaluate the replacement of the original Cashco air pressure regulating valves in the Weld Channel and Containment Penetration Pressurization System (WCCPPS) with stainless steel regulators. The existing regulators were carbon steel. The modification also replaced the existing rupture discs with safety relief valves to provide improved over-pressure protection for system components.

Summary of Safety Evaluation

The original Cashco valves installed as WCCPPs air-side pressure regulating valve, PS-PCV-1193, 1195, 1197, and 1199 were replaced with equivalent Cashco stainless steel regulators. The replacement regulators meet the requirements of the original purchase specification. To protect the air regulators, new filters were installed and existing 175 psi rupture discs, PC-1181-1,2,3,4 downstream of each regulator were replaced with 82 psi safety relief valves. The change provides greater assurance that system components will be protected from over-pressurization and that a WCCPPS zone will not be rendered inoperable as a result of a transient pressure spike.

1997 ANNUAL REPORT

MMP 96-3-273 FP, REV. 0

RETIREMENT OF FIRE ALARM ZONE 7

Description and Purpose

The purpose of this modification was to address concerns identified in LER 95-013-00 which required a revision to the IP3 Operational Specifications section 3.5.4 to delete the reference to duct mounted smoke detectors installed in the Control Room HVAC return duct work. Also, to meet commitments made by NYPA to the NRC in letter IN-95-084-01.

Summary of Safety Evaluation

A Nuclear Safety Evaluation (NSE) 95-3-310 was performed to evaluate the LER and recommended the removal of the Zone 7 Fire Protection System exhaust duct smoke detectors installed in the Control Room HVAC return duct work. The modification retired in place fire alarm Zone 7 detectors, conduit, cables, and a panel alarm module. The intent of the NRC staff position is to have the licensee provide early warning detection in safety related areas, cabinets and consoles. The intent is met in the Control Room by the existing area-wide smoke detection system installed at the ceiling and detectors installed in the cabinets. The removal of the duct mounted smoke detectors does not affect the ability of the area-wide detection system, is consistent with the National Fire Codes and fully meets the intent of the NRC staff position on detection.

1997 ANNUAL REPORT

NSE 96-3-275 ED, REV. 1

REALIGNMENT OF POWER FEED TO AUXILIARY COMPONENT COOLING WATER PUMPS (ACCW) 32 & 33

Description and Purpose

The purpose of this evaluation was to evaluate the change in power supplies to the Auxiliary Component Cooling Water (ACCW) Pumps 31 and 32 to ensure compliance to IP3 Technical Specification, accident analysis and design basis documentation.

Summary of Safety Evaluation

The modification MMP 96-3-275 ED realigned the power supplies to ACCW Pumps 31 and 32 so they are powered from the same power train as the Recirculating Pumps (RP) they support (RP31 and RP32 respectively). The modification was done to bring the plant design into agreement with the technical specification limiting condition for operation . The realignment of the power supplies removes the condition where the single failure criteria was not being met. The original design did not consider a single failure of the power supply to the motor control centers when certain ACCW Pumps are taken out of service.

1997 ANNUAL REPORT

NSE 96-3-278 RCS, REV. 0

PRESSURIZER SPRAY VALVES BELLOWS SEAL REMOVAL

Description and Purpose

This NSE evaluated the replacement of the bellows seal assembly on Pressurizer Spray Valves RC-PCV-455A/B. The original bellows seals were added to decrease Reactor Coolant leakage however, were not reliable and required replacement every other cycle to prevent fatigue. They have since been replaced with a single live-load packing set which eliminates the bellows seals and bonnet extension.

Summary of Safety Evaluation

Live load packing has become a standard practice in the industry for reducing leakage. EPRI studies and industry experience have shown that live-loaded packing can provide assurance of stem leakage less than that assumed in the FSAR (Section 6.7.2.3) for the life of the packing without adjustment.

1997 ANNUAL REPORT

MMP 96-3-279 TS, REV. 0

REPLACEMENT OF
VERTICAL SCREEN WASH PUMPS 31, 32, AND 33

Description and Purpose

The purpose of this evaluation was to provide justification for the replacement of the existing vertical turbine type screen wash pumps with submersible type pumps.

Summary of Safety Evaluation

The original three Ingersoll Rand vertical turbine type screen wash pumps and motors were replaced with submersible type pumps and motors. The new pumps and motor assemblies were installed on the existing foundation bolts. The pump discharge pipe is at the existing elevation attached to the piping system using the existing flexible connector. There was no change in the system design operating pressure or flow characteristics since the replacement pumps were purchased to the same performance requirements as the originals. The modification also provided for closure of Temporary Modifications 96-04967-03 (pump 32), 96-03850-02, and 91-33826-01 (pump 33). The new pumps provide increased reliability over the originals which had an extensive history of maintenance problems caused by excessive shaft vibration.

1997 ANNUAL REPORT

MOD 96-3-289, REV. 0

FUEL TRANSFER CART DRIVE SYSTEM UPGRADE

Description and Purpose

This evaluation addressed the replacement of the existing control system with an industry proven reliable Programmable Logic Controller (PLC) based controls, which provides automatic and manual control of the system.

Summary of Safety Evaluation

This modification was performed to improve the reliability and safety of the transfer cart system. It was concluded that this modification does not involve an unreviewed safety question. The upgrade improves reliability and maintainability of the fuel transfer cart system.

1997 ANNUAL REPORT

NSE 96-3-394 FP, REV. 1

CHANGES TO THE FIRE PROTECTION PROGRAM AS DESCRIBED IN THE FSAR

Description and Purpose

The purpose of this evaluation was to review proposed changes to the IP3 Fire Protection Program as described in the FSAR.

Summary of Safety Evaluation

The changes involve clarifications to the IP3 Fire Protection Program as described in the FSAR and standardization in the manner in which the fire detection system and types of fire detectors are described in the FSAR. The changes do not degrade the Fire Protection Program or adversely affect the ability to achieve and maintain safe shutdown.

1997 ANNUAL REPORT

NSE 96-3-447 IVSW, REV. 0

ALTERNATE LOCATION FOR DEPRESSURIZING THE IVSW/SGBD HEADER

Description and Purpose

This NSE Converted Temporary Modification (TM) TM 96-01553-04 to a permanent modification. The purpose of this TM is to provide a convenient and safe method for venting/depressurizing the Isolation Valve Seal Water (IVSW) lines associated with the Steam Generator Blowdown (SGBD) and SGBD sampling systems.

Summary of Safety Evaluation

This TM was designed and installed with the intent that it could become a permanent modification. It complies fully with all IVSW system design criteria and does not adversely affect any other safety related equipment.

1997 ANNUAL REPORT

NSE 96-3-463, REV. 1

OUTAGE EQUIPMENT HATCH

Description and Purpose

The purpose of the Outage Equipment Hatch is to replace the hemispherical containment door used in normal operating conditions. The Outage hatch will be utilized during plant shutdowns when the equipment door is removed. Its function is similar to the Equipment Door - to seal containment in the event of an accident in which radioactive material is released. The OEH was designed to assist outage planning and ultimately shorten refueling/maintenance outages by allowing the large containment equipment hatch door to remain open for the entire outage duration.

Summary of Safety Evaluation

The OEH does not directly influence the operation of any system whose malfunction or misuse could lead to a design basis accident.
It is a passive device that serves to isolate the Vapor Containment Building atmosphere from the environment, consistent with the requirements of Technical Specification 3.8.A.1. Therefore, installation of the Outage Equipment Hatch does not increase the probability of occurrence of an accident evaluated in the safety analysis report.

NSE 96-3-504 SWS, REV. 2

INSTRUMENT AIR CLOSED COOLING SUPPLY & RETURN SERVICE WATER PIPING UPGRADE

Description and Purpose

The purpose of this evaluation was to provide justification for the replacement of Service Water piping and valves with a 6% Molybdenum (Mo) austenitic stainless steel to improve the corrosion resistance.

Summary of Safety Evaluation

The piping and valves that were replaced include the 2-1/2" and 3" supply and return piping from the 10" headers in the transformer yard into the Control Building up to and from the connections to the Instrument Air Cooling Water Heat Exchangers 31 and 32. This piping has required extensive maintenance due to corrosion and was a concern for potential future leaks in the 15' elevation of the Control Building (480V Switchgear Area). The modification did not change the overall design of the Service Water supply to the Instrument Air Cooling Water Heat Exchangers. The replacement piping and valves provide the same function, pressure class, nominal diameter, and pressure retaining capability as the original components.

1997 ANNUAL REPORT

MMP 96-3-518 ARM, REV. 0

REMOVAL FROM SERVICE OF R-10 AUXILIARY FEEDWATER PUMP BUILDING AREA RADIATION MONITOR

Description and Purpose

The purpose of this modification was to evaluate and retire Radiation Monitor R-10 in place. This included the disconnection of the unit power supply, detectors, indicators, recorders, alarm and computer input points.

Summary of Safety Evaluation

The radiation monitor, R-10 meets the Regulatory Guide 1.97 requirements and measures radiation levels adjacent to the vapor containment following a loss-of-coolant accident. The monitor assists operations personnel to determine if the accident consequences are either more or less serious than predicted. High range containment radiation monitors R-25 and R-26 have also been installed in containment and R-10 acts as a back up to these. These redundant monitors meet the requirements of Regulatory Guide 1.97 and are used to measure the area radiation fields in the Vapor Containment. This evaluation and analysis concluded that the current function of R-10 can be met through a combination of the use of more accurate detectors, R-25 and R-26, for accident consequences analysis; and by other locally positioned detectors R-62A through D or through the use of portable survey instruments for direct field measurements as the primary means of monitoring the radiation levels inside the building.

1997 ANNUAL REPORT

NSE 96-3-523 AMA, REV. 0

USE OF ETHANOLAMINE (ETA) AS A SECONDARY pH ADDITIVE

Description and Purpose

The purpose of this evaluation was to evaluate the use of ethanolamine for secondary pH control as an alternative to morpholine.

Summary of Safety Evaluation

The use of ETA as a secondary pH additive was evaluated as an alternative to morpholine. The use of ETA has reduced iron corrosion product transport to the steam generators at many pressurized water reactors. The use of ETA has been endorsed by the EPRI Secondary Guidelines Committee but had to be evaluated at IP3 prior to permanent implementation because its benefits and side effects are site specific. The evaluation concluded that the use of ETA at IP3 was acceptable.

1997 ANNUAL REPORT

NSE 96-3-535 IVSW, REV. 0

DELETION OF FSAR DESIGN REQUIREMENT TO INSTALL DISCHARGE LINE FOR RELIEF VALVE IV-1475 OF IVSWS

Description and Purpose

The purpose of this evaluation was to evaluate the impact of deleting a FSAR design requirement (Fig. 6.5-1) of the Isolation Valve Seal Water System (IVSWS).

Summary of Safety Evaluation

The evaluation provides the analysis to justify the elimination from the FSAR (Fig. 6.5-1) the design requirement to install a discharge line from relief valve IV-1475 routed to the nearest radioactive drain. The review and analysis prove that, based on single failure analysis, the safety relief valve IV-1475 would not be exposed to a backpressure from the lines to which nitrogen is injected and as a result no radioactive release is possible.

1997 ANNUAL REPORT

NSE 97-3-028 SWS, REV. 2

SERVICE WATER PIPING MATERIAL

Description and Purpose

This NSE evaluated the acceptability of replacing the cement lined, carbon and stainless steel Service Water piping and valves with an unlined austenitic stainless type of alloy which is more resistant to corrosion.

Summary of Safety Evaluation

The new piping material retains the function, pressure class and pressure retaining capability of the Service Water System and has improved overall reliability due to its increased corrosion resistance.

1997 ANNUAL REPORT

NSE 97-3-044 CCW, REV. 0

REPLACEMENT OF FLANGE AND ELBOW IN LINE 325
DOWNSTREAM OF VALVE AC-803

Description and Purpose

The purpose of this evaluation was to evaluate the replacement of the flange and elbow in Component Cooling Water (CCW) Line 325.

Summary of Safety Evaluation

Minor Modification 97-3-044 CCW replaced the existing flange and elbow in line 325 immediately downstream of Valve AC-803 with hardened carbon steel components which provide better erosion resistance in a cavitation environment. The replacement components are in conformance with the Class 152N specification requirements except that the material was treated with a diffusion alloying process to increase the internal surface hardness. The piping configuration, material properties, and operating conditions were not affected by the modification.

1997 ANNUAL REPORT

NSE 97-3-062 CVCS, REV. 0

REVISE FSAR SECTION 6.2.3 PER ACTS 17482

Description and Purpose

The purpose of this evaluation was to justify changes to the FSAR which provide clarification to Section 6.2.3 related to the Reactor Coolant Pump (RCP) seal return line from the containment penetration to the Volume Control Tank (VCT). Clarification that data represented on Table 6.7-2 is for information only, and administrative correction to Section 1.3.5 related to type description of the Chemical and Volume Control System (CVCS), RCP seal injection and normal charging header containment isolation valves.

Summary of Safety Evaluation

The evaluation provided justification to revise the appropriate sections of the FSAR. The changes are administrative in nature to reflect actual performance of the leakage identification and reduction program and the actual containment isolation valve type for the CVCS charging header / RCP seal injection system.

1997 ANNUAL REPORT

NSE 97-3-077 VCHA, REV. 0

CHANNELIZATION OF CONTAINMENT ISOLATION FOR HYDROGEN MONITORS

Description and Purpose

The purpose of this evaluation was to address concerns documented in Deviation Event Report (DER) 97-0173 in regard to the configuration of the containment isolation valves for the vapor containment (VC) atmospheric hydrogen concentration monitoring cabinet (HCMC) system, and the automatic closure signals for these valves from the Engineered Safeguards System (ESS).

Summary of Safety Evaluation

The evaluation demonstrated that the provision of specified administrative controls for testing and operation of the HCMC System satisfies the design requirements. In addition, the evaluation served to correct a number of discrepancies in the FSAR which were identified during the review of DER 97-0173.

1997 ANNUAL REPORT

NSE 97-3-080 CET, REV. 0

CHANGE FSAR TO REFLECT MOVEABLE INCORE DETECTOR SYSTEM AS PRIMARY MEANS OF MALPOSITIONED ROD DETECTION

Description and Purpose

This NSE justified using the Moveable Incore Detector System as the primary means of detecting dropped and misaligned rods instead of the Core Exit Thermocouple System (CET).

The Moveable Incore System measures neutron flux in 61 axial points along the core. Dropped rods are detected by lower neutron flux. The actual position can be determined with very good accuracy by the axial points.

The CET system is based on changes in coolant temperature due to decreases in power caused by neutron absorption from the dropped rod. This method is not as reliable due to crossflow and mixing of the coolant as it moves up through the core. This mixing effect lowers the exit temperature and as a result slight deviations are difficult to detect. In addition, decreases in exit temperature are directly proportional to the misalignment so if misalignment is slight, detection may be difficult.

Summary of Safety Evaluation

The Moveable incore detector system is a more accurate means of detecting dropped or misaligned rods and is justified as the primary indicator. The CET, although not as accurate can be used as another indicator of a grossly misaligned rod.

1997 ANNUAL REPORT

NSE 97-3-083, REV. 0

PZR REVISION TO PRESSURIZER RELIEF TANK VENTING CONFIGURATION

Description and Purpose

The purpose of this evaluation was to review the change in Pressurizer Relief Tank (PRT) venting configuration (valve position) to allow easier access and prevent component damage during venting operations while in plant shutdown. The evaluation also resolved Reactor Coolant System (RCS) configuration control issue identified in Deviation Event Report 96-2242.

Summary of Safety Evaluation

Valve number RC-517 must be manipulated in accordance with PRT venting procedures SOP-RCS-7 and SOP-SS-3. The location of the valve is on top of the PRT immediately below the floor grating. El. 68' in the Vapor Containment Building. Access to the valve is very difficult and there has been a historical problem with damage to other components in the area during the course of PRT ventilation. This evaluation provides the analysis and justification to change the valve configuration to provide ventilation through an alternate path using valves that are easily accessible.

1997 ANNUAL REPORT

NSE 97-3-085 RHR, REV. 0

FSAR CHANGE TO SECTION 6.2 REGARDING CAPABILITY

Description and Purpose

The purpose of this evaluation was to justify plant configuration and identify required FSAR changes to resolve a discrepancy regarding motor operated valves AC-MOV-730 and 731.

Summary of Safety Evaluation

The revision to the FSAR to properly reflect the design and operation of AC-MOV-730 and 731 and their related peripheral equipment does not adversely affect the ability of the plant to prevent and mitigate accidents. With the physical design improvements and the operational changes implemented since the last revision to the affected portion of the FSAR, this is considered an enhancement to the protection scheme designed to preclude overpressurization of the Residual Heat Removal system piping.

1997 ANNUAL REPORT

NSE 97-3-108 CM, REV. 0

FSAR CLARIFICATION OF FSB HOOK TESTING

Description and Purpose

The purpose of this evaluation was to clarify the testing of the Fuel Storage Building (FSB) crane hooks as described in the IP3 FSAR section 9.5.

Summary of Safety Evaluation

The FSAR section 9.5 states: "The lifting hooks were purchased, fabricated and load tested to manufacturers standard. The hooks were ultrasonically tested to detect any flaws. *The hooks are again tested at specified periods and prior to all lifts.* "The FSAR change provides clarification as to the type of testing that should be performed in accordance with NRC commitments and industry standards for the FSB crane 40 and 5 ton hooks.

1997 ANNUAL REPORT

NSE 97-3-133 N2, REV. 0

INSTALLATION OF A PERMANENT N2 SUPPLY TO PRESSURIZER FOR DRAIN DOWN

Description and Purpose

The purpose of this evaluation was to evaluate changes to plant configuration and operation as a result of Minor Modification Package (MMP) 97-3-133 N2 which relocated and upgraded the Nitrogen supply source used to determine Pressurizer water level during reduced inventory operations.

Summary of Safety Evaluation

The modification removed the existing Nitrogen supply line which consisted of tubing that was connected when required via a flange downstream of a normally closed Safety Injection Accumulator vent valve. It was replaced with a permanently mounted line connected to line #3001 of the Nitrogen System near the Pressurizer on the 95 foot level of the Containment Building. This eliminates an extended run and the necessity to control the installation and removal of the flange. The modification also replaced undesirable portions of the system with an improved design.

1997 ANNUAL REPORT

NSE-97-3-140 VCV, REV. 0

FSAR CLARIFICATION OF FCU HEPA FILTER TESTING

Description and Purpose

This NSE clarified FSAR section 6.4, regarding testing requirements for HEPA filters used in the containment fan cooler system.

A statement as to the type of testing used for the HEPA filters on the FCUs was unnecessary because it is covered in the Reg. Guide 1.52 and was therefore deleted.

Summary of Safety Evaluation

This change is administrative in nature and has no safety significance. No changes to the design, testing or operation of HEPA filters have been made.

1997 ANNUAL REPORT

NSE 97-3-143 RHR, REV. 0

DRAINING THE REFUELING CAVITY TO THE RWST

Description and Purpose

The purpose of this evaluation was to evaluate and demonstrate that the safety related refueling activity of partially draining the refueling cavity using the Residual Heat Removal (RHR) system and the Refueling Cavity Drain Pump does not adversely affect overall plant safety and that it does not adversely impact the overall system operating criteria contained in the licensing basis documents.

Summary of Safety Evaluation

The evaluated activity is safety related and only occurs during refueling with the plant in cold shutdown and the Reactor Coolant System (RCS) de-pressurized and vented. The cavity draindown does not prohibit the RHR system from performing its intended function. The Refueling Water Storage Tank (RWST) is vented to atmosphere and flow is permitted under controlled conditions which ensures that the low pressure piping of the suction piping from the RWST to the Safety Injection (SI) pumps is not challenged. Interlocks associated with valve SI-MOV-883 are intended to prevent inadvertent opening and loss of RCS inventory during periods of normal RHR shutdown cooling.

1997 ANNUAL REPORT

CLAS 97-3-157 FP, REV. 0

CLASSIFICATION OF FIRE SUPPRESSION AND DETECTION SYSTEM FOR BUILDINGS AND STRUCTURES OUTSIDE OF PROTECTED AREA

Description and Purpose

The purpose of this classification was to classify the Fire Protection system for buildings and structures outside the protected area. Previous classification 86-3-02 Rev.1 classified all permanently installed fire protection systems to Category M. It did not specifically describe systems, components, and features outside the protected area at IP3.

Summary of Safety Evaluation

As a result of this classification, all associated NYPA owned and maintained components and equipment referenced in the classification and located outside of the protected area at IP3 were classified as Non-Category I. Those portions of the fire protection systems are disassociated from portions of the system that protect safety related / safe shutdown equipment either by isolation or because they are independent. Adverse conditions will not impact the ability of the plant to achieve and maintain shutdown in the event of a fire.

NSE 97-3-159 SWS, REV. 0

DOCUMENT CHANGE RESOLUTION FOR REMOVAL OF RETIRED HEAT TRACE FOR SERVICE WATER PIPING TO CIRC WATER PUMP BEARINGS/SEALS

Description and Purpose

The purpose of this evaluation was to evaluate the effect of Document Change Resolution (DCR) 100134125 on the plants' Licensing Basis. The DCR revised the Service Water flow diagram to reflect the removal of heat tracing from the 2" Service Water piping to the Circulating Water Pump bearings/seals following implementation of Type One Change DC 96-3-510.

Summary of Safety Evaluation

DC 96-3-510 SWS replaced the 2" Service Water supply piping to the Circulating Water Pump bearings and seals. The piping was heat traced but the heat tracing was retired in place by minor modification MMP 93-3-333 which installed a low point drain to facilitate draining the line in lieu of the heat tracing. When the piping was replaced per the DC, it was re-insulated and the heat tracing was not re-installed since it was no longer in service and served no function.

1997 ANNUAL REPORT

NSE 97-3-164 PZR, REV. 0

PRESSURIZER HEATER BACKUP GROUP 31-555KW
DOCUMENTATION

Description and Purpose

This NSE corrected a discrepancy in the FSAR concerning the wattage of the pressurizer heater backup group #31, which is remotely controlled from outside of the Control Room. The FSAR indicated the wattage was 485KW instead of 555KW.

Summary of Safety Evaluation

This change makes the FSAR consistent with the DBD and other plant documents. Since there is no physical change, the activity does not create the possibility of an accident.

1997 ANNUAL REPORT

NSE 97-3-167 ADM, REV. O

CHANGES TO SECTION 17.2 OF IP3 FSAR AND QA PROGRAM MANUAL

Description and Purpose

This NSE removed specific references to Quality Assurance Procedures (QAPs) in FSAR Figure 17.2-5. The new figure lists procedures applicable to QA on a more generic basis in lieu of listing each specific QAP.

Summary of Safety Evaluation

Eliminating specific reference to QAPs is an administrative change and has no safety significance.

1997 ANNUAL REPORT

MMP 97-3-176, REV. 1

STEAM GENERATORS SNUBBERS ELIMINATION

Description and Purpose

The purpose of this modification was to eliminate all 24 Hydraulic Snubbers used in the Steam Generators (SG) supporting system.

Summary of Safety Evaluation

The objective of the performed analysis was to demonstrate that 100% of the snubbers [six per steam generator (SG) or twenty four total] can be removed from the plant without any significant impact on the remainder of the plant.

1997 ANNUAL REPORT

MMP 97-3-178 CET, REV. 0

RETIREMENT OF CET'S IN PLACE

Description and Purpose

This modification covered the retirement in place of certain Core Exit Thermocouples (CET's). The CET's were unrecoverable because of the length of the pigtail left after repairs or were broken during removal. Thermocouples with pigtails less than 4.00 inches were cut off flush with the conduit pressure fitting and capped with a swagelok stainless steel pressure cap.

Summary of Safety Evaluation

This modification of retiring in place ten (10) previously qualified and non-qualified CET's and capping conduits, will have no effect on previously analyzed scenarios in the FSAR, since the CET's do not perform any safety or automatic function(s) and are used for indication only. By placing the Westinghouse provided (solid) Swagelok cap on the conduit guide tubes, RCS integrity is maintained as the cap is rated for a higher pressure that normal and accident RCS pressures. The broken off thermocouples can not migrate to the core. After evaluation of this modification, it was determined that no unreviewed safety question exists.

1997 ANNUAL REPORT

NSE 97-3-183 RHR, REV. 0

PLANT COOLDOWN - HOT TO COLD SHUTDOWN - INITIATE RHR PRIOR TO 250 DEGREES

Description and Purpose

The purpose of this NSE was to evaluate the safety significance of placing the Residual Heat Removal (RHR) system in service between 250F and 350F during normal shutdown operations.

Summary of Safety Evaluation

The FSAR description for RHR is applicable for a normal cooldown as well as event recovery. As stated in the FSAR placing RHR in service at 350F is well within the system design. This guidance allows the system to be operated in compliance with analyzed conditions (in operation at approximately 350F), or delayed to minimize the thermal transient on the system. There is no impact on the safety analysis, Technical Specifications, or Operational Specifications.

1997 ANNUAL REPORT

MMP 97-3-193 RW, REV. 0 AND NSE 97-3-248, REV. 1

UPGRADE OF SERVICE WATER FCU PIPE SUPPORTS FOR WATER HAMMER EVENT

Description and Purpose

This modification upgraded several pipe supports for the Containment Fan Cooler Service Water Supply and Return piping. Pipe supports upgrades are required to enable the system to withstand the resulting water hammers which occur either from condensate void collapse or column following restart of the service water pumps.

Summary of Safety Evaluation

The upgraded FCU system supports will ensure that the integrity of its engineered safety features to perform their required function is structurally maintained for the effects of a Loss-of-offsite and/or a Loss-of-coolant-Accident per IP-3 USFAR requirements. As a result of this modification the upgraded service water system and containment fan cooler units will meet the system design and operability requirements for water hammer events as discussed in NRC Generic letter No. 96-06.

1997 ANNUAL REPORT

MMP 97-3-207 RCC, REV. 0

ARPI & CRDM CONNECTOR REPLACEMENT

Description and Purpose

This modification covered the Westinghouse replacement and testing of the Analog Rod Position Indication (ARPI) and Control Rod Drive Mechanism (CRDM) cable connectors, and the replacement of the ARPI top plates. The ARPI & CRDM connectors demonstrated problems which increased the disconnect / reconnect time spent on the head, and personnel exposure.

Summary of Safety Evaluation

The CRDM and ARPI System do not provide any protection grade type function. The new cables, connectors and top plates will provide the same function as the original cabling connections. The effect of the equipment change is to enhance disconnect/reconnect of the CRDMs and the ARPI cabling system. This modification does not impact any safety system function necessary to bring the plant to a safe shutdown or to mitigate release of radioactive material.

1997 ANNUAL REPORT

MMP 97-3-208 RM, REV. 0

R-62 RANGE AND INDICATION

Description and Purpose

The purpose of this modification was to correct the detection and the indication range for R-62 (Main Steam Line Radiation Monitor) specified in the Final Safety Analysis Report (FSAR), and to correct Modification 91-3-101 with regard to the local analog indicator and alarm assemblies used for the main steam line channels.

Summary of Safety Evaluation

This modification modified the analog output range from the R-62 main steam line radiation monitor, and delineated which main steam line activity channels have a local analog indicator and alarm assemblies.

1997 ANNUAL REPORT

NSE 97-3-211 RHR, REV. 0

EVALUATION FOR DE-ENERGIZATION OF RHR SUCTION LINE ISOLATION VALVES AC-MOV-730 AND AC-MOV-731 AT COLD SHUTDOWN & REVISE CAPACITY OF AC-1836 RELIEF VALVE

Description and Purpose

The purpose of this NSE was to evaluate the de-energizing of AC-MOV-730 and AC-MOV-731 and to revise the IP3 UFSAR to reflect the current configuration of the plant, specifically for operation when the Reactor Coolant System (RCS) is in Cold Shutdown and Refueling Conditions with the RCS de-pressurized and vented. Additionally, this NSE also corrected an incorrect statement in the FSAR regarding the capacity of the RHR suction relief valve AC-1836.

Summary of Safety Evaluation

Revision and clarification of the affected sections of the FSAR to reflect the configuration changes and controls regarding the operation of the RHR Suction Line Isolation Valves AC-MOV-730 and 731 and their related peripheral equipment and controls, will not adversely affect the ability of the plant to prevent and mitigate accidents. This change in operation is considered an enhancement to the protection scheme by preventing an inadvertent loss of Residual Heat Removal (RHR) due to a spurious closure of 730 or 731.

1997 ANNUAL REPORT

NSE 97-3-212 FHS, REV. 0

EVALUATION OF USE OF GUIDE STUDS FOR REACTOR VESSEL DISASSEMBLY

Description and Purpose

The Purpose of this NSE was to evaluate the use of guide studs in the disassembly of the Reactor Vessel during refueling operations.

Summary of Safety Evaluation

The use of two reactor vessel guide studs instead of three guide studs does not impact the safe operation or safe shutdown of Indian Point 3. The guide studs are only used during refueling operations and are not required to prevent or mitigate the effects of design basis accidents.

1997 ANNUAL REPORT

NSE 97-3-227 VCHVP, REV. 0

OPERATION OF THE PAB/VC EXHAUST FAN WITHOUT PURGE SUPPLY FAN

Description and Purpose

The purpose of this evaluation was to evaluate the effect of the implementation of Temporary Modification (TM) 94-02469-00 on the safe operation of the plant. The TM removed the purge supply fan from service during the Vapor Containment (VC) purge.

Summary of Safety Evaluation

The installation of the TM allowed the PAB/VC Purge Exhaust fan to run and purge the VC without running the containment purge supply fan. This ensured a negative pressure inside the VC preventing the VC atmosphere from venting outside the VC relative to the PAB and outside. This mode of operation is only used at cold shutdown. Removing the interlock for the purge supply fan does not cause any equipment damage. The interlock and the fans are not required to mitigate or prevent any accident previously described in the FSAR.

1997 ANNUAL REPORT

CLS 97-3-258 CC, REV. 0

RECLASSIFICATION OF 3" LINE #127 FROM 8" LINE #328
TIE-IN TO 3" VALVE AC-629

Description and Purpose

The purpose of this classification was to re-classify a portion of 3" line #127 [Spent Fuel Pit (SFP) Filter Return Line] from Q.A. Non-Category I, Seismic Class III to Q.A. Category I, Seismic Class II. Upgrading Valve AC-629 from Seismic III to Seismic II provided the capability of isolation following a seismic event which will preclude partial drainage of the SFP and loss of Spent Fuel Pit Cooling (SPFC).

Summary of Safety Evaluation

This classification was required to ensure that the line is isolable following an Operating Basis Earthquake (OBE), to preclude partial drainage of the spent fuel Pit and loss of Spent Fuel Cooling. As a result of the implementation of this classification, the Spent Fuel Pit Cooling loop will be assured of being capable of achieving its safety related function of cooling the spent fuel assemblies, following a seismic event and resultant downstream line break of interfacing non-safety related piping. Thus, this classification of Spent Fuel Pit Filter Return line #127 does not involve an unreviewed safety question.

1997 ANNUAL REPORT

NSE 97-3-269 HC, REV. 0

HYDROGEN MONITOR IN-SERVICE REQUIREMENT

Description and Purpose

This document evaluated the result of placing the Hydrogen Monitor in service in excess of 30 minutes following initiation of safety injection using controls in the control room and field actions.

Summary of Safety Evaluation

This evaluation addressed the Operations concerns that operator work load and event mitigation actions impact the inability to meet the license commitment to place the hydrogen monitoring system in service within 30 minutes of a safety injection actuation.

This evaluation concluded that there are no unreviewed safety questions associated with placing the Hydrogen Monitor in service within 7 hours following a LOCA or MSLB inside containment. Operation of the system is performed in a time frame that will support placing the hydrogen recombiners in service and maintaining containment hydrogen concentration less than flammable limit.

1997 ANNUAL REPORT

NSE 97-3-270 HVAC, REV. 0

CCR/AC FILTER SYSTEM NEW FLOWS & DIFF PRESS

Description and Purpose

The purpose of this NSE was to define the flow rate and DP (differential pressure) for the Control Room Heating Ventilation & Air Conditioning (CR HVAC) filtration system.

Summary of Safety Evaluation

The new flow rates are based on licensing and current plant design basis criteria for 31 days of hands off operation of the CR booster fans in the 10% incident mode. This will set the acceptance criteria for the surveillance test requirements.

1997 ANNUAL REPORT

MMP 97-3-293, REV. 0, NSE 97-3-293, REV. 1

STEAM GENERATOR BLOWDOWN WATER HAMMER PREVENTION

Description and Purpose

The purpose of this modification was to prevent/mitigate water hammer in the Steam Generator Blowdown (SGBD) system.

Summary of Safety Evaluation

This modification cross connects the SGBD with the SGBD sample system. This cross connection will use the high pressure sample system fluid to re-fill the voided space in the SGBD system after closure of the SGBD containment isolation valves. Filling the voided space in the SGBD line will mitigate/prevent water hammer. The SGBD systems are not used in the mitigation of the consequences of an accident.

1997 ANNUAL REPORT

NSE 97-3-299 SI, REV. 0

REPLACEMENT OF SI-MOV-899A (MO) MOTOR

Description and Purpose

The purpose of the Modification was to replace the motor and fuses associated with valve SI-MOV-899A. This modification was performed due to non availability of the same size motor.

Summary of Safety Evaluation

This modification addressed the replacement of the motor on Residual Heat Loop Discharge Stop valve SI-MOV-899A with a lower Horse Power (10.3HP) and larger full load amperage (15.1A) than the originally installed motor of 10.5 HP having 13.8A full load amperage. The Safety evaluation concludes that the subject Motor Operated Valve (MOV), with replaced motor, will not affect safe plant operation or have impact on the environment.

1997 ANNUAL REPORT

NSE 97-3-309 H2, REV. 0

VCT HYDROGEN SYSTEM EVALUATION

Description and Purpose

This Nuclear Safety Evaluation was performed to ensure that the removal of HS-PCV-1041 from the nuclear hydrogen supply manifold does not result in an Unreviewed Safety Question.

Summary of Safety Evaluation

The removal of valve HS-PCV-1041 from the nuclear hydrogen supply manifold will not have a negative impact on the FSAR, Technical Specifications or any other design basis document. Additionally, the removal of this valve does not have any impact on plant operability and performance.

1997 ANNUAL REPORT

CLS 97-3-311 ED, REV. 0

UPGRADE THE QA CLASSIFICATION OF BATTERY CHARGERS 31,32,33, AND 35 FROM CATEGORY M TO CATEGORY I. BATTERY CHARGER 34 WILL REMAIN CATEGORY M

Description and Purpose

To upgrade the QA classification of battery charger 31,32,33, and 35 from category M to Category I. Battery Charger 34 will remain Category M, since it is not required to perform a QA category I function.

Summary of Safety Evaluation

This re-classification of Battery Chargers 31,32,33, and 35 does not increase the probability of occurrence or consequences of an accident or malfunction of structures, systems or components important to safety previously evaluated in the FSAR because upgrading the QA classification of the above battery chargers will make the DC system more reliable.

1997 ANNUAL REPORT

NSE 97-3-315 CS, REV. 0

FSAR CHANGES ON CONTAINMENT SPRAY AND SODIUM HYDROXIDE CANCEL SIGNAL

Description and Purpose

This Nuclear Safety Evaluation (NSE) addressed the implied operator action to prevent Sodium Hydroxide (NaOH) injection after Containment Spray (CS) initiation signal. In addition spray cancellation was clarified and the FSAR Section 6.3.2 updated accordingly.

Summary of Safety Evaluation

This activity clarifies statements on spray cancellation and termination of CS initiation signals. This NSE will delete a paragraph from the CS FSAR related to the instructions provided to operators to cancel NaOH addition to containment spray following a containment spray initiation signal. These FSAR changes do not increase the probability of occurrence of an accident previously evaluated in the FSAR.

1997 ANNUAL REPORT

NSE 97-3-325 RCS, REV. 0

STEAM GENERATOR TUBE RUPTURE RE-EVALUATION

Description and Purpose

The purpose of the Nuclear Safety Evaluation (NSE) was to demonstrate that the inclusion of Reactor Coolant System (RCS) charging flow and an operator action time for terminating break flow, greater that 30 minutes assumed in the current Indian Point 3 (IP-3) steam generator tube rupture (SGTR) analysis of record, will not adversely affect the results of the current analysis of record and, therefore, will not adversely affect safe plant operation.

Summary of Safety Evaluation

The safety significance of adding charging flow to IP-3 SGTR analysis, and considering a greater operator action time required to terminate a SGTR has been evaluated as required according to the criteria of 10 CFR 50.59, and does not represent an unreviewed safety question.

1997 ANNUAL REPORT

NSE 97-3-329 SI, REV. 0

CHANGE BIT ISOLATION VALVES POSITIONS FROM NORMALLY CLOSED TO OPEN

Description and Purpose

This evaluation justifies maintaining the Boron Injection Tank (BIT) isolation valves in the open position during normal operations.

Summary of Safety Evaluation

This evaluation justifies the acceptability of changing the normal positions of valves SI-1835A&B (BIT outlet isolation) and SI-1852A&B (BIT inlet isolation) from closed to open. These changes do not alter the function or the BIT flowpath, and do not challenge the integrity of any affected components. The functional elimination of the Boron Injection Tank has eliminated the design requirements for these valves to be closed during normal power operations. This change increases the safety of the plant by eliminating the reliance of the active safety function of these valves to open in order to achieve the required performance of the high head safety injection system.

1997 ANNUAL REPORT

NSE 97-3-338 ED, REV. 1

EVALUATION OF DC SYSTEM REQUIREMENTS AND BATTERY CHARGERS DESIGN BASIS

Description and Purpose

This NSE involved revising the design basis of Battery Chargers 31,32, and 33 to ensure that a continuous source of power will be available to the 125VDC and 118VAC safety-related loads subsequent to the 2-hour duty cycle of the Station Batteries. This was accomplished by reclassifying Battery Chargers 31,32,33 and 35 as Q.A. Category I (safety related) so that they can be credited with performing their safety function and by evaluating the recently revised procedures to ensure that appropriate measures are taken so that a continuous source of power is available to the 125VDC and 118VAC safety-related loads following a Loss of Offsite Power (LOOP), Safety Injection Signal or Seismic event.

Summary of Safety Evaluation

An evaluation of the design basis functions of the 125VDC system resulted in the determination of additional safety functions required to be supported by Battery Chargers 31,32,33, and 35. This necessitated a classification upgrade from Q.A.Category M Seismic Class III to Q.A. Category I Seismic Class I. The basis for the upgrade has been adequately justified and substantiated in Classification 97-3-311. Procedural revisions required to ensure that power is available to the 125 VDC and 118VAC safety related equipment have been completed to provide the assurance that the Battery Chargers will fulfill their intended function. These activities will increase the reliability of the 125VDC and 118 VAC systems.

1997 ANNUAL REPORT

NSE 97-3-351 SI, REV. 0

EVALUATION OF ES-1.3 AND FSAR CHANGES DUE TO EXTENT OF CONDITION REVIEW

Description and Purpose

This NSE evaluates the changes to EOP ES-1.3, and supports changes to the FSAR as necessary to update or correct the FSAR to correspond to the current analyses.

Summary of Safety Evaluation

This NSE reviewed the changes to procedure ES-1.3, "Transfer to Cold leg Recirculation" to ensure that design basis requirements are met for various plant systems, structures and components.

1997 ANNUAL REPORT

NSE 97-3-353 RW, REV. 0

PLUGS FOR CONTAINMENT RECIRCULATION FAN MOTOR COOLER SERVICE WATER DRAIN VALVES SWN-631, 632, 633, 634, AND 635

Description and Purpose

To review the use of threaded plugs on the downstream side of Containment Recirculation Fan (CRF) Motor Cooler Service Water Drain Valves SWN-631 through SWN-635. Threaded plugs would limit leakage from the valves.

Summary of Safety Evaluation

The use of plugs on the outlet side of the drain valves will provide double isolation to the drains from the CRF Motor Coolers. In addition, the plugs will provide a level of foreign material exclusion (FME) to the downstream side of the drain valve internals and prolong the service life of the valves.

1997 ANNUAL REPORT

NSE 97-3-356 ED, REV. 0

REVISE FSAR DESCRIPTION FOR 125 VOLT DC SYSTEM

Description and Purpose

The purpose of this evaluation was to review FSAR Sections 1.3.2, 8.2.2, and 8.2.3 125 VDC System Descriptions, and specify changes which reflect the "as-built" configuration.

Summary of Safety Evaluation

Specific changes to FSAR Sections 1.2.3, 8.2.2, and 8.2.3 were identified and initiated. These changes did not reflect an unreviewed safety question and served to improve the quality of plant configuration.

1997 ANNUAL REPORT

NSE 97-3-359 SMPL, REV. 0

FSAR UPDATE FOR
STEAM GENERATOR SAMPLING SYSTEM

Description and Purpose

Addresses concerns, with regard to the position of Steam Generator Blowdown Sample Containment isolation Valves BD-PCV-1223 through 1226, and BD-PCV-1223 A through 1226A, by proposing to change the SGBD Sampling System's Containment Isolation Valves position in FSAR Tables 5.2-2, and 5.2-3, and Figure 5.2-15 from closed to open. This proposed change would make the FSAR consistent with OPs Procedures.

Summary of Safety Evaluation

Changing the normal position of the Steam Generator Blowdown Sample Containment Isolation Valves from "closed" to "open" in the appropriate places in the FSAR does not result in an unreviewed safety question.

1997 ANNUAL REPORT

NSE 97-3-360 STR/VC, REV. 0

EVALUATION OF R.6 TO AP-39, ASME CODE REPAIR

Description and Purpose

To determine the safety significance of implementing revision 6 of AP-39, the IP3 ASME Code section XI Repair/Replacement Program. The implementation is required by amended (NRC) regulations which are intended to ensure that critical areas of containment are routinely inspected to detect and take corrective action for defects that could compromise the containment structural integrity.

Summary of Safety Evaluation

The new requirements ensure that the components classified "MC" and "CC," and the piping/components which are part of the Containment System are repaired, replaced, and inspected in accordance with an effective Program to manage potentially age-related Containment degradation. AP-39, revision 6 does not change the design, or prevent the system from performing its function, and does not increase the probability of an accident or malfunction. Also, the procedure revision does not create any new accident or equipment malfunction which was not evaluated.

1997 ANNUAL REPORT

NSE 97-3-364 RW/SWS, REV. 0

ADMINISTRATIVE CHANGE TO FSAR PAGE 6.4-21

Description and Purpose

Resistance Temperature Detectors (RTDs) installed on the fan cooler service water outlet lines, and a RTD installed on the inlet line were removed from the Environmental Qualification (EQ) Program (and the EQ Master Equipment List). The RTD's provide ΔT indication on the Critical Function Monitoring System (CFMS). The FSAR identifies these as EQ Components.

In order to facilitate an FSAR change which will bring the FSAR description into conformance with the existing plant configuration, verify the heat removal capability of the RTD's .

Summary of Safety Evaluation

The RTD's are located in a mild environment. Removal from the EQ Program is justified. Thus an FSAR change is required.

1997 ANNUAL REPORT

NSE 97-3-365 MS, REV. 0

32 AUXILIARY BOILER FEED PUMP TURBINE DRAINS

Description and Purpose

Justify the acceptability of changing the normal position of drain (high pressure) valves for the 32 Auxiliary Feedwater Pump Turbine from cracked open to closed, and leaving one drain (low pressure) valve cracked open. This configuration change is intended to ensure a suitable environment in the ABFP Room following automatic pump starts which result in the room filling with steam.

Summary of Safety Evaluation

Closing the 32 Auxiliary Boiler Feedpump Turbine Drain Valves MS-111, MS-112-1, and MS-113, and cracking open MS-114 does not result in an unreviewed safety question. Thus the new alignment is acceptable.

1997 ANNUAL REPORT

NSE 97-3-373 H2, REV. 0

USING H2 TRUCK AS PRIMARY SOURCE OF H2

Description and Purpose

To demonstrate the acceptability of using a contractor (via an H_2 truck) to provide the primary source of hydrogen during normal operations. Acceptability would allow the existing H_2 storage system to be isolated and abandoned in place. (Note: a temporary modification is in place).

Summary of Safety Evaluation

Using an H_2 Trailer does not degrade the Security Plan, QA Program, or Fire Protection System, and no unreviewed safety question is raised.

1997 ANNUAL REPORT

NSE 97-3-377 WCCPP, REV. 0

EVALUATION OF SINGLE FAILURE ANALYSIS FOR WCCPP

Description and Purpose

The purpose of this task was to evaluate the single failure design basis of the Weld Channel and Containment Penetration Pressurization (WCCPP) System and correct the FSAR.

Summary of Safety Evaluation

Resulting from the evaluation, revisions to the FSAR were indicated as follows: 1) revise FSAR Section 6.6.3; 2) Delete Table 6.6-3; 3) Clarify page 14.3-32, and Tables 14.3-15, 16, and 17; and 4) minor changes to page 6.1-1.

1997 ANNUAL REPORT

NSE 97-3-378 CS, REV. 0

CHANGE FSAR CONTAINMENT SPRAY PUMP MARGIN, AND TABLE 6.2-13

Description and Purpose

The purpose of this evaluation was to resolve discrepancy in FSAR NPSH values for the Containment Spray Pumps.

Summary of Safety Evaluation

IP3-CALC-CS-02590, Revision 0, a new Containment Spray Pump NPSH calculation was prepared. As a result, the following FSAR changes were required: 1) Update Section 6.3.3, and Table 6.2-13; and 2) Remove the sample calculation of the CS Pump NPSH from the attachment to Table 6.2-13.

1997 ANNUAL REPORT

NSE 97-3-379 WCCPP, REV. 0

FSAR REVIEW OF WCCPP GAUGE ACCURACY

Description and Purpose

WCCPP pressurized zones are equipped with local gauges whose accuracy are ±2 % of the full scale reading. The FSAR specifies gauge accuracy of ±0.5 % of the full scale reading. The purpose of this evaluation was to reconcile the differences in accuracy.

Summary of Safety Evaluation

Acceptability of ±2.0 % of the full scale reading for the subject gauges was demonstrated. Based on the evaluation, the appropriate FSAR revision was initiated.

1997 ANNUAL REPORT

NSE 97-3-381 SI, REV. 0

FSAR UPDATE OF VALVES SI-MOV-885A & 885B

Description and Purpose

The purpose of this evaluation was to address a proposed change in the normal position of these valves to "closed" in FSAR Table 5.2-3. FSAR Table 5.2-3 indicates the normal position of containment isolation valves SI-MOV-885A, SI-MOV-885B, SI-MOV-888A, and SI-MOV-888B is "locked shut." Operation Department Procedures (SOP-CB-1, SOP-CB-1A, SOP-CB-11, COL-SI-1, COL-RHR-1, Drawing Number 9321-F-27503, Revision 36 (Flow Diagram Safety Injection System, Sheet 2), and FSAR Table 5.2-2 indicates the normal position of these valves is "normally closed."

Summary of Safety Evaluation

Changing the normal position of containment isolation valves SI-MOV-885A, SI-MOV-885B, SI-MOV-888A, and SI-MOV-888B from "locked shut" to "closed," in FSAR Table 5.2-3 is acceptable.

1997 ANNUAL REPORT

NSE 97-3-382 SI, REV. 0

ALT LOW TO HIGH RECIRCULATION FLOWPATH LEAKAGE

Description and Purpose

The use of an alternate recirculation flowpath is described in FSAR Table 6.2-8. During post-LOCA recirculation an alternate path (if normal path is lost) requires the RHR pumps to draw suction from the Containment Sump and deliver a suction supply to the 32 SI Pump through valves SI-MOV-883, and SI-898. This alternate flowpath could be in service for an extended period of time and would (or could) contain highly radioactive fluids. However, this flowpath was not considered for external recirculation loop leakage and was not included in FSAR Section 6.2.3 which describes NYPA's Program to identify and reduce leakage from systems outside containment which would (or could) contain highly radioactive fluids. The purpose of this evaluation was to justify adding this flowpath to FSAR Section 6.2.3.

Summary of Safety Evaluation

The subject flowpath (including all branches up to their respective isolation valves) will be included in the program for leak detection and reduction, and appropriate FSAR changes will be initiated.

1997 ANNUAL REPORT

NSE 97-3-387 RW, REV. 0

REVISION OF HYDRO REQUIREMENT FOR SERVICE WATER PUMPS

Description and Purpose

The purpose of this evaluation was to update an erroneous statement in FSAR Section 9.6.1 with regard to the hydrostatic test pressure of Service Water Pumps.

Summary of Safety Evaluation

The Main Service Water Pumps were replaced with Ingersoll-Rand pumps in 1989. The new pump specifications required hydrostatic test pressures which were less than the former pumps. Since the test pressures specified for the new pumps more than meets the ASME requirements, the FSAR will be revised accordingly.

1997 ANNUAL REPORT

NSE 97-3-389 SMPL, REV. 0

USE OF COMPRESSION FITTINGS IN THE SAMPLE SYSTEM

Description and Purpose

Compression fittings are used throughout the sampling system. The FSAR confines the use of these fittings to the vicinity of the sampling sink. The purpose of this document was to evaluate the use of compression fittings in the sampling system at locations other than the vicinity of the sampling sink.

Summary of Safety Evaluation

The use of compression fittings throughout the sampling system is justified. The FSAR will be revised accordingly.

1997 ANNUAL REPORT

NSE 97-3-394 SI, REV. 0

EVALUATION OF FSAR CHANGES OF RECIRCULATION PUMP MOTORS

Description and Purpose

This Nuclear Safety Evaluation (NSE) justified the revision to FSAR Section 6.2.2 which added more descriptive information regarding the design of Safety Injection Recirculation Pump Motors. This NSE also removed the reference to Section 6.4.2 which described the Fan Cooler Motors Safety Injection Recirculation Pump Motors as being identical.

Summary of Safety Evaluation

These changes are editorial in nature. They do not reflect the discovery of any design discrepancies nor do they affect the actual plant configuration.

1997 ANNUAL REPORT

NSE 97-3-398 ADMIN, REV. 0

IP3 DESIGN ENGINEERING DEPARTMENT
REORGANIZATION

Description and Purpose

The purpose of this evaluation was to review a proposed change to the Design Engineering Department organization.

Summary of Safety Evaluation

The changes were Administrative only, involving redistribution of responsibilities and associated title changes. There is no safety significance initiated by these changes.

1997 ANNUAL REPORT

NSE 97-3-401 RF, REV. 0

REVISION TO EDG SWS FLOW CONTROL VALVES DESCRIPTION IN FSAR

Description and Purpose

The purpose of this evaluation was to evaluate (clarify) the flow rates through EDG Service water flow control valves FCV-1176, and FCV-1176A which are reported in FSAR Section 9.6.1.

Summary of Safety Evaluation

FSAR Section 9.6.1 states that each (FCV-1176, and FCV-1176A) is sized for a maximum flow of 1500 gpm. The condition at which this flow would be expected is not provided. However, a recent flow balance test was able to supply all components with 1520 gpm. Therefore it is appropriate to delete the 1500 gpm flow rate from the FSAR. Since this flow rate pertains to one static point in the operating system no value will be included and Section 9.6.1 will be revised.

1997 ANNUAL REPORT

NSE 97-3-403 ADMIN, REV. 0

ORGANIZATIONAL CHANGES

Description and Purpose

Evaluate proposed changes to the NYPA organization.

Summary of Safety Evaluation

The changes included creation and deletion of management positions, management title changes, and reassignment of position responsibilities and reporting relationships. The proposed changes are administrative and do not affect Plant equipment or operating conditions. FSAR Sections 12.1, and 17.2 will be revised to reflect the changes.

1997 ANNUAL REPORT

NSE 97-3-406 RW, REV. 0

REMOVAL OF 8 SWITCH SWS ACTION TO RECIRCULATION

Description and Purpose

This evaluation justified the elimination of the 8 switch sequence (from FSAR Section 9.6.1) in conjunction with Service Water System actions during switchover from the injection to the recirculation phases following a design basis accident.

Summary of Safety Evaluation

The 8 switches involve the set-up of the SI and CCW systems for long-term recirculation, a process which is independent of the service water system, as long as the service water system maintains its role as the ultimate heat sink for these systems. No safety concerns were identified, therefore, mention of the 8 switch sequence will be removed from FSAR Section 9.6.1.

1997 ANNUAL REPORT

NSE 97-3-408 HVAC, REV. 0

CLOSURE OF 1 RETURN REGISTER CCR SYSTEM

Description and Purpose

The purpose of this document was to evaluate the safety significance of operating the Control Room Air Conditioning System (CRACS) with the air flow through one of the Control Room (CR) return registers restricted by throttling or closing its volume damper.

Summary of Safety Evaluation

Setting the volume damper position between partially open and fully closed will result in improved air distribution in the CR and utilization of the CRACS. FSAR Figure 9.9-1 will be changed to reflect this activity.

1997 ANNUAL REPORT

NSE 97-3-430 COND, REV. 0

FSAR REVISION TO SECTION 10.2.26, PAGE 12.2.-26

Description and Purpose

The purpose of this document was to evaluate operation of the Condensate Polishing System (CPS) with the Condensate Booster Pumps secured or in standby.

Summary of Safety Evaluation

Securing the Condensate Booster pumps when placing the Condensate Polisher in service has been determined to have no significant effects on the balance of plant. The proposed FSAR change provides the option of feeding the low pressure feedwater heaters with or without the Condensate Booster Pumps.

1997 ANNUAL REPORT

NSE 97-3-446 ADMIN, REV. 0

REORGANIZATION OF EMERGENCY RESPONSE AND SUPPORT RECOVERY

Description and Purpose

Evaluate proposed changes to the NYPA organization responsible for recovery activities following an emergency. Specifically, the Headquarters Emergency Response/Recovery Organization (HERRO) will be eliminated. Its functions will be performed by a de-centralized Recovery Organization.

Summary of Safety Evaluation

The changes provide flexibility to effectively distribute, assign, and utilize the resources needed for recovery activities. HERRO will be eliminated. The changes are administrative in nature and do not affect plant equipment or operating conditions.