

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 736.8001



Robert J. Barrett
Site Executive Officer

August 20, 1997
IPN-97-111

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
License No. DPR-64
Radioactive Effluent Release and Waste Disposal Semi-Annual
Report for the period January 1, 1997 through June 30, 1997

Dear Sir:

Enclosed is the Semi-Annual Report of Radioactivity in Solid Wastes and Releases of Radioactive Material in Liquid and Gaseous Effluents for Indian Point 3 as required by the Environmental Technical Specifications Section 5.3.2.1. The enclosed report covers the period January 1, 1997 through June 30, 1997 for Indian Point 3 and would include those releases from Indian Point 2 resulting from processing liquid waste from Indian Point 3 if this pathway was utilized. During this reporting period, no waste was transferred from Indian Point 3 to Indian Point 2.

Attachment I contains the commitment the Authority is making in this submittal. If you have any questions please contact Steve Sandike at (914) 736-8455.

Very truly yours,


R. J. Barrett
Site Executive Officer
Indian Point 3 Nuclear Power Plant

IE48 11

Report
Attachment

cc: See next page

9708290266 970630
PDR ADDCK 05000286
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IP3 Records Center

Effluent and Waste Disposal
Semi-Annual Report
January 1, 1997 - June 30, 1997

Facility Indian Point 3
Licensee New York Power Authority

This information is provided in accordance with the requirements of Regulatory Guide 1.21. The numbered sections of this report reference corresponding sections of the subject Regulatory Guide, pages 10 to 12.

A. Supplemental Information

1. Regulatory Limits

Indian Point 3 is presently subject to limits on radioactive waste releases that are set forth in sections 2.3.1, 2.3.2, 2.3.3, 2.4.1, 2.4.2, 2.4.3 and 2.4.4 of Appendix B to Docket No. 50-286 Amended Facility Operating License entitled "Environmental Technical Specification Requirements Part II Radiological Environmental" (ETSR). The percentages of the technical specification limits reported in Tables 1A and 2A are the percent of the quarterly limits specified in the ETSR. If more than one limit applies to the release, the most restrictive limit is reported.

2. Maximum Permissible Concentration

a) Fission and Activation Gases

The quarterly dose resulting from release of fission and activation gases is calculated in accordance with the methodology stated in the Offsite Dose Calculation Manual (ODCM). The specific isotopes listed in Table 1C are used to determine the effective dose factors for the time period.

b/c) Iodines, Tritium and Particulates

The quarterly organ dose limit for Iodine 131, tritium and particulates with half-lives greater than eight days is calculated in accordance with the methodology stated in the ODCM.

d) Liquid Effluents

The quarterly dose limit for liquid isotopic releases is calculated in accordance with the methodology stated in the ODCM. The concentration limit for noble gases dissolved in liquid releases is calculated based upon a maximum permissible concentration of 2.00E-4 uCi/ml as required by section 2.3.1.A of the ETSR.

3. Average Energy

The average energies (\bar{E}) of the radionuclide mixtures in releases of fission and activation gases were as follows:

1st Quarter $\bar{E}_\beta = 1.40\text{E-}01$ Mev/dis $\bar{E}_\gamma = 4.71\text{E-}02$ Mev/dis

2nd Quarter $\bar{E}_\beta = 1.39\text{E-}01$ Mev/dis $\bar{E}_\gamma = 4.91\text{E-}02$ Mev/dis

4. Measurements and Approximations of Total Radioactivitya) Fission and Activation Gases

Analysis of effluent gases has been performed in compliance with the requirements of Table 3.4-1 of the ETSR. In the case of isolated tanks (batch release), the total activity discharged is based on an isotopic analysis of each batch with the volume of gas in the batch corrected to standard temperature and pressure.

Vapor Containment purge discharges that are less than 150 hours/quarter in duration have been treated as batch releases and Vapor Containment pressure relief discharges have been treated as continuous releases (> 500 hrs/year and as defined in NUREG 0133, Section 3.3). At least one complete isotopic concentration analysis of containment air is performed monthly. This analysis is used in conjunction with a process monitor to obtain the isotopic mixture and quantification of each pressure relief. Isotopic analyses for each Vapor Containment purge are taken prior to and during the purge. This information is combined with the volume of air in each discharge to calculate the quantity of activity released from these discharges.

The continuous building discharges are based on weekly samples of ventilation air for isotopic content. This information is combined with total air volume discharged and the process radiation monitor readings to determine the quantity of activity from continuous discharges.

b/c) Iodines and Particulates

Iodine-131 and particulate releases are quantified by collecting a continuous sample of ventilation air on a TEDA impregnated, activated charcoal cartridge and a glass-fiber filter paper. These samples are changed weekly as required in Table 3.4-1 of the ETSR and the concentration of isotopes found by analysis of these samples is combined with the volume of air discharged during the sampling period to calculate the quantity of activity discharged.

For other iodine isotopes the concentration of each isotope is determined monthly on a 24-hour sample. The concentration of the isotopes found by analysis is combined with the volume of air discharged during the sampling period to calculate the quantity of activity discharged.

d) Liquid Effluents

A sample of each batch discharge is taken and an isotopic analysis is performed in compliance with requirements specified in Table 3.3-1 of the ETSR. This isotopic concentration data is combined with the information on volume discharged to determine the amount of each isotope discharged.

Proportional composite samples of continuous discharges are taken and analyzed in compliance with Table 3.3-1 of the ETSR. This concentration data is combined with the volume discharged to calculate the total activity discharged.

5. Batch Releases

a) Liquid

	<u>1997</u>	
	<u>1st Quarter</u>	<u>2nd Quarter</u>
Number of Batch Releases	61	40
Total Time Period Batch Releases (min)	6008	4255
Maximum " " " " "	153	190
Average " " " " "	99	106
Minimum " " " " "	60	78
Average Stream Flow (cfs)	Note: *	Note: *

Note:*

This information is obtained from the Department of the Interior, U.S. Geological Survey, for the Hudson River. Due to the delays in obtaining this data from the governmental agency, flows will be submitted as they become available.

b) Gaseous

Number of Batch Releases	21	17
Total Time Period Batch Releases (min)	2249	1906
Maximum " " " " "	518	330
Average " " " " "	107	112
Minimum " " " " "	14	15

6. Abnormal Releases

- a) Liquid
None
- b) Gaseous
None

7. Radiological Environmental Technical Specifications

The Radiological Environmental Technical Specifications (RETS) require reporting of prolonged outages of effluent monitoring equipment (Sections 2.1.C and 2.2.B) and significant changes in the land use census, Radiological Environmental Monitoring Program (REMP), or exceeding the total curie content limitations in outdoor tanks (Sections 2.8.A, 2.8.B, 2.7.C and 2.3.4.B).

During this reporting period, the following required Technical Specification Effluent Monitoring equipment was out of service (OOS) for periods greater than 30 consecutive days:

Equipment OOS	Period OOS	Reason for Out of Service Condition
R-19 Steam Generator Blowdown	From May 16, 1997 To Jun 30, 1997 (46 days)	Insufficient sample flow while Steam Generators in cold wet layup during cold shutdown plant condition

Indian Point 3
EFFLUENT AND WASTE DISPOSAL
SEMI-ANNUAL REPORT

B. GASEOUS EFFLUENTS
FIRST AND SECOND QUARTERS, 1997

TABLE 1A

EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT (Jan - Jun 1997)

GASEOUS EFFLUENTS - SUMMATION OF ALL RELEASES

	UNIT	QUARTER 1st	QUARTER 2nd	EST. TOTAL ERROR %
A. Fission & Activation Gases				
1. Total Release	Ci	1.19E+02	1.30E+02	±25
2. Average release rate for period	uCi/sec	1.53E+01	1.65E+01	
3. Percent of technical spec. limit	%	1.99E-01	2.16E-01	
B. Iodines				
1. Total Iodine - 131	Ci	4.85E-05	2.32E-04	±25
2. Average release rate for period	uCi/sec	6.24E-06	2.95E-05	
C. Particulates				
1. Total release with T½ > 8 days	Ci	0.00E-00	2.46E-06	±25
2. Average release rate for period	uCi/sec	0.00E-00	3.13E-07	
3. Gross alpha radioactivity	Ci	<2.70E-07	<3.10E-07	
D. Tritium				
1. Total release	Ci	7.25E-01	7.31E-01	±25
2. Average release rate for period	uCi/sec	9.32E-02	9.30E-02	
E. Percent of Tech Spec Limit Iodines, Particulate with T½ > 8days, & Tritium				
	%	3.00E-02	1.41E-01	±25

TABLE 1C
EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT (Jan - Jun 1997)
GASEOUS EFFLUENTS-GROUND RELEASES

Nuclides Released	Unit	CONTINUOUS MODE		BATCH MODE		
		1st Quarter	2nd Quarter	1st Quarter	2nd Quarter	
1) Fission Gases						
Krypton (Kr) 85m	Ci	4.02E-03	3.42E-04	4.13E-02	3.56E-03	
Krypton (Kr) 85	Ci	3.64E-01		2.65E-00	6.71E-01	
Krypton (Kr) 87	Ci			4.22E-03		
Krypton (Kr) 88	Ci	2.18E-03		3.69E-02		
Xenon (Xe) 131m	Ci	1.29E-01	2.27E-02	1.51E-00	4.51E-01	
Xenon (Xe) 133m	Ci	1.25E-01	1.67E-02	7.77E-01	2.51E-01	
Xenon (Xe) 133	Ci	2.22E+01	9.34E+01	9.02E+01	3.31E+01	
Xenon (Xe) 135m	Ci					
Xenon (Xe) 135	Ci	1.57E-01	1.98E-00	4.20E-01	1.21E-01	
Xenon (Xe) 138	Ci					
Argon (Ar) 41	Ci	4.16E-02	2.14E-02	5.08E-02	4.01E-04	
TOTAL FOR PERIOD	Ci	2.30E+01	9.55E+01	9.57E+01	3.46E+01	
2) Iodines						
Iodine (I) 131	Ci	4.85E-05	2.32E-04			
Iodine (I) 133	Ci					
Iodine (I) 135	Ci					
TOTAL FOR PERIOD	Ci	4.85E-05	2.32E-04	0.00E-00	0.00E-00	

TABLE 1C
 EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT (Jan - Jun 1997)
 GASEOUS EFFLUENTS - GROUND RELEASES

Nuclides Released	Unit	CONTINUOUS MODE		BATCH MODE	
		1st Quarter	2nd Quarter	1st Quarter	2nd Quarter
3) Particulates					
Antimony (Sb) 125	Ci				
Barium (Ba) 133	Ci				
Cadmium (Cd) 109	Ci				
Cerium (Ce) 139	Ci				
Cerium (Ce) 141	Ci				
Cerium (Ce) 144	Ci				
Cesium (Cs) 134	Ci				
Cesium (Cs) 137	Ci		1.63E-06		
Cobalt (Co) 57	Ci				
Cobalt (Co) 58	Ci				
Cobalt (Co) 60	Ci				
Chromium (Cr) 51	Ci				
Niobium (Nb) 95	Ci		8.39E-07		
Strontium (Sr) 89	Ci				
Strontium (Sr) 90	Ci				
Tin (Sn) 113	Ci				
TOTAL	Ci	0.00E-00	2.46E-06	0.00E-00	0.00E-00

Indian Point 3

EFFLUENT AND WASTE DISPOSAL

SEMI-ANNUAL REPORT

C. LIQUID EFFLUENTS

FIRST AND SECOND QUARTERS, 1997

TABLE 2A

EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT (Jan - Jun 1997)

LIQUID EFFLUENTS - SUMMATION OF ALL RELEASES

	UNITS	QUARTER 1st	QUARTER 2nd	EST. TOTAL ERROR %
A. Fission and activation products				
1. Total release (not including tritium, gases, alpha)	Ci	3.61E-02	3.91E-02	±25
2. Average diluted concentration during period	uCi/ml	1.59E-10	2.27E-10	
B. Tritium				
1. Total release	Ci	1.33E+02	3.90E+01	±25
2. Average diluted concentration during period	uCi/ml	5.85E-07	2.27E-07	
C. Dissolved and entrained gases				
1. Total release	Ci	9.98E-02	3.63E-02	±25
2. Average diluted concentration during period	uCi/ml	4.38E-10	2.11E-10	
D. Gross alpha radioactivity				
1. Total release	Ci	<5.31E-05	<4.11E-05	±25
E. Volume of waste released (prior to dilution)				
	liters	1.33E+06	9.14E+05	±10
F. Volume of dilution water used during period				
	liters	2.28E+11	1.72E+11	±10
G. Percent of liquid effluent limit				
	%	9.98E-02	4.34E-02	±25

TABLE 2B
LIQUID EFFLUENT AND WASTE DISPOSAL SEMI-ANNUAL REPORT (Jan - Jun 1997)

Nuclides Released	Unit	CONTINUOUS MODE		BATCH MODE	
		1st Quarter	2nd Quarter	1st Quarter	2nd Quarter
Sodium (Na) 24	Ci			8.10E-05	2.58E-05
Chromium (Cr) 51	Ci			1.10E-04	5.04E-03
Manganese (Mn) 54	Ci			8.77E-05	4.24E-04
Iron (Fe) 55	Ci			3.09E-03	4.85E-03
Iron (Fe) 59	Ci				1.27E-04
Cobalt (Co) 58	Ci			7.04E-03	6.71E-03
Cobalt (Co) 60	Ci			3.10E-03	5.44E-03
Nickel (Ni) 63	Ci			3.95E-03	3.72E-03
Strontium (Sr) 85	Ci			2.05E-05	5.25E-06
Strontium (Sr) 90	Ci				6.22E-06
Zirconium (Zr) 95	Ci				1.97E-05
Niobium (Nb) 95	Ci				2.77E-05
Antimony (Sb) 124	Ci			4.06E-03	5.15E-03
Antimony (Sb) 125	Ci			2.42E-03	4.16E-03
Iodine (I) 131	Ci			5.87E-05	7.60E-05
Cesium (Cs) 134	Ci			4.08E-03	1.15E-03
Cesium (Cs) 137	Ci			8.02E-03	2.14E-03
Lanthanum (La) 140	Ci				8.12E-06
TOTAL FOR PERIOD	Ci	0.00E-00	0.00E-00	3.61E-02	3.91E-02

Nuclides	Unit	CONTINUOUS MODE		BATCH MODE	
		1st Quarter	2nd Quarter	1st Quarter	2nd Quarter
Argon (Ar) 41	Ci				3.51E-06
Krypton (Kr) 85	Ci			4.68E-03	1.20E-03
Xenon (Xe) 135	Ci			1.06E-04	1.41E-05
Xenon (Xe) 133m	Ci			4.49E-04	5.16E-05
Xenon (Xe) 131m	Ci			3.55E-03	3.22E-04
Xenon (Xe) 133	Ci			9.10E-02	3.47E-02
TOTAL DISSOLVED AND ENTRAINED GASES	Ci			9.98E-02	3.63E-02

Indian Point 3
EFFLUENT AND WASTE DISPOSAL
SEMI-ANNUAL REPORT

D. SOLID WASTE
FIRST AND SECOND QUARTERS, 1997

TABLE 3
EFFLUENT AND WASTE DISPOSAL SEMIANNUAL REPORT
January 1 - June 30, 1997
SOLID WASTE SHIPMENTS

A. SOLID WASTE SHIPPED OFFSITE FOR BURIAL OR DISPOSAL (Not irradiated fuel)

1. Type of Waste	Unit	6 Month Period			Est. Total Error, %
		Class A	Class B	Class C	
a. Spent resins, filter sludges, etc.	m ³	0	0	0	±25
	Ci	0	0	0	
b. Dry compressible, contam. equipment for burial	m ³	0	0	0	±25
	Ci	0	0	0	
c. Irradiated Components	m ³	0	0	0	±25
	Ci	0	0	0	
d. Other: Dry compressible, contaminated equip. for volume reduction at offsite facility	m ³	2.76E+2	0	0	±25
	Ci	1.39E+0	0	0	

2. **Estimate of major nuclide composition (by type of waste)**

NUCLIDE	UNIT	Dry Vol. Red	Resin	Resin
		CLASS A	CLASS A	CLASS B
C-14	%	6.00		
Fe-55	%	33.72		
Co-60	%	21.86		
Ni-59	%	0.38		
Cs-137	%	1.66		
Pu-241	%	0.10		
Ni-63	%	36.28		

Percentages of nuclides and total activities are based on a combination of direct measurements and scaling for non-gamma emitting nuclides.

3. **Solid Waste Disposition**

Number of <u>Shipments</u>	Mode of <u>Transport</u>	<u>Destination</u>	
4	Truck	SEG, Oak Ridge TN	for volume reduction
1	Truck	Hake, Memphis TN	for volume reduction
3	Truck	A.E., Oak Ridge TN	for volume reduction

4. **Containers Shipped**

<u>Container</u>	<u>Number</u>	<u>Class A</u>		<u>Class B</u>		<u>Class C</u>	
		<u>Solid. Media</u>	<u>Number</u>	<u>Solid. Media</u>	<u>Number</u>	<u>Solid Media</u>	
<i>For Burial:</i>							
Poly HIC	0	N/A	0	N/A	0	N/A	
Drums	0	N/A	0	N/A	0	N/A	
Steel Liner	0	N/A	0	N/A	0	N/A	
Crates	0	N/A	0	N/A	0	N/A	
<i>For Volume Reduction:</i>							
SeaLand Cont.	6	N/A	0	N/A	0	N/A	
Crate	16	N/A	0	N/A	0	N/A	
Six Pack	0	N/A	0	N/A	0	N/A	
Drums	71	N/A	0	N/A	0	N/A	

Indian Point 3

EFFLUENT AND WASTE DISPOSAL

SEMI-ANNUAL REPORT

E. RADIOLOGICAL IMPACT ON MAN

(Not required to be submitted with this report)

Indian Point 3
EFFLUENT AND WASTE DISPOSAL
SEMI-ANNUAL REPORT

F. METEOROLOGICAL DATA

(Not required to be submitted with this report)

Indian Point 3

EFFLUENT AND WASTE DISPOSAL

SEMI-ANNUAL REPORT

G. OFFSITE DOSE CALCULATION MANUAL OR
PROCESS CONTROL PROGRAM OR LAND USE CENSUS LOCATION CHANGES

FIRST AND SECOND QUARTERS, 1997

(Neither the Process Control Program nor the ODCM were changed during this reporting period. No new locations for dose calculations and/or environmental monitoring were identified by the land use census.)

List of Commitments

Number	Commitment	Due
IPN-97-111-01	Average stream flow information is obtained from the Department of the Interior, U.S. Geological Survey, for the Hudson River. Due to the delays in obtaining this data from the governmental agency, flows will be submitted as they become available.	When data becomes available

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Senior Vice President and
Chief Nuclear Officer

January 6, 2000
IPN-00-001
JPN-00-001

David L. Meyer
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Washington, DC 20555-0001
ATTN: Rulemakings and Adjudications Staff

SUBJECT: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
**Comments on Revised Criteria for
Post Accident Sampling System**

REFERENCES: 1.) Federal Register, November 24, 1999, Volume 64, Number 226,
pages 66213-66214.
2.) NEI letter to NRC, L. Hendricks to D. Meyer, "Revised Criteria for
Post Accident Sampling Systems November 24, 1999, 64 Fed.
Reg. 66213," dated January 5, 2000.

The Authority has reviewed the request for comment regarding the revised criteria for the Post Accident Sampling Systems (PASS) published in the referenced Federal Register notice. This notice states that the NRC is considering the endorsement of topical reports written by the Westinghouse and Combustion Engineering Owners Groups. These reports provide justification for the elimination of the PASS from the licensing basis and would allow licensees to remove their commitments to maintain the PASS.

The Authority supports the NRC's decision to endorse these topical reports. Both the Westinghouse Owners Group and Combustion Engineering Owners Group have determined that there is no decrease in emergency planning effectiveness as a result of eliminating the regulatory requirements to maintain a dedicated PASS. The Authority Emergency Planning personnel for both IP3 and JAF have spoken with state and local emergency response organizations about the referenced Federal Register notice to ensure its potential impact is understood.

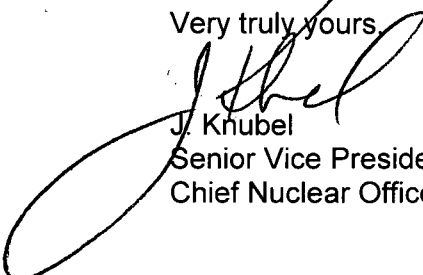
003678996

Add: J.O'Brien

The Authority has reviewed the Nuclear Energy Institute's (NEI) comment letter on this Federal Register Notice and supports NEI's position on this issue.

There are no commitments made by the Authority in this letter. If you have any questions, please contact Ms. C. Faison.

Very truly yours,



J. Khubel
Senior Vice President and
Chief Nuclear Officer

U.S. Nuclear Regulatory Commission
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