

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 736.8001



Robert J. Barrett
Site Executive Officer

April 9, 1999
IPN-99-039

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
License No. DPR-64
Monthly Operating Report for March 1999

Dear Sir:

The attached monthly operating report, for the month of March 1999, is hereby submitted in accordance with Indian Point 3 Nuclear Power Plant Technical Specification 6.9.1.4.

The Authority is making no commitments in this letter.

Very truly yours,

A handwritten signature in cursive script, appearing to read 'Robert J. Barrett', with a long horizontal line extending to the right.

Robert J. Barrett
Site Executive Officer
Indian Point 3 Nuclear Power Plant

cc: See next page

9904230145 990331
PDR ADOCK 05000286
R PDR

230019

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FE24

Attachments

cc: Mr. Hubert J. Miller
Regional Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406-1415

Resident Inspector's Office
Indian Point Unit 3
U.S. Nuclear Regulatory Commission
P.O. Box 337
Buchanan, NY 10511

U.S. Nuclear Regulatory Commission
ATTN: Director, Office of Information Resource Management
Washington, D.C. 20555

INPO Records Center
700 Galleria Parkway
Atlanta, Georgia 30339-5957

OPERATING DATA REPORT

DOCKET NO. 50-286
 DATE: 4-1-99
 COMPLETED BY: T. Orlando
 TELEPHONE NO: (914) 736-8340
 LETTER NO: IPN-99-039
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OPERATING STATUS

1. Unit Name: Indian Point No. 3 Nuclear Power Plant
2. Reporting Period: March 1999
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report Give Reasons:

9. Power Level to Which Restricted, If Any (Net MWe): 510 Net MWe
10. Reasons for Restrictions, If Any: # 31 & # 32 Main Boiler Feedwater Pumps were alternately out of service during the period of March 12 - 17

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	<u>744</u>	<u>2160</u>	<u>198,097</u>
12. Number Of Hours Reactor Was Critical	<u>711.35</u>	<u>2127.35</u>	<u>115,119.41</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>683.68</u>	<u>2099.68</u>	<u>112,502.99</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1,855,869</u>	<u>6,110,620</u>	<u>320,164,760</u>
17. Gross Electrical Energy Generated (MWH)	<u>620,190</u>	<u>2,051,590</u>	<u>101,483,945</u>
18. Net Electrical Energy Generated (MWH)	<u>598,971</u>	<u>1,985,424</u>	<u>97,681,468</u>
19. Unit Service Factor	<u>91.9</u>	<u>97.2</u>	<u>56.8</u>
20. Unit Availability Factor	<u>91.9</u>	<u>97.2</u>	<u>56.8</u>
21. Unit Capacity factor (Using MDC Net)	<u>83.4</u>	<u>95.3</u>	<u>52.1*</u>
22. Unit Capacity Factor (Using DER Net)	<u>83.4</u>	<u>95.3</u>	<u>51.1</u>
23. Unit Forced Outage Rate	<u>8.1</u>	<u>2.8</u>	<u>27.5</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each): Refueling Outage 10 scheduled to commence September 10, 1999. Estimated duration 45 days.
25. If Shut Down At End Of Report Period. Estimated Date of Startup: _____

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

* Weighted Average

AVERAGE DAILY UNIT POWER LEVEL

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MONTH March 1999

DAY	AVERAGE DAILY POWER	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	982	17	518
2	982	18	912
3	982	19	978
4	981	20	980
5	978	21	979
6	980	22	979
7	981	23	979
8	981	24	979
9	587	25	980
10	0	26	980
11	0	27	980
12	345	28	980
13	498	29	979
14	508	30	977
15	509	31	977
16	508		

INSTRUCTIONS: On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March 1999

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR 3	LICENSEE EVENT REPORT #	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
1	980309	F	59.92	B	3	1999-003	CJ	INSTRUT	UNIT TRIP OCCURRED WHILE RETURNING NO. 33 REACTOR COOLANT PUMP FLOW TRANSMITTER TO SERVICE FOLLOWING INSTALLATION OF A TEMPORARY MODIFICATION.
2	990312	F	N/A	A	N/A	N/A	CH	VALVEX C	UNIT LOAD RESTRICTED TO APPROXIMATELY 58% REACTOR POWER DUE TO MAINTENANCE ON NO. 31 & 32 MAIN BOILER FEEDWATER PUMP (MBFP) DISCHARGE VALVES. THE PUMPS WERE ALTERNATELY OUT OF SERVICE.

1
 F: Forced
 S: Scheduled

2
 Reason:
 A- Equipment
 B- Maintenance or Test
 C- Refueling
 D- Regulatory Restriction
 E- Operator Training & Licensee Examination
 F- Administrative
 G- Operational Error
 H- Other (Explain)

3
 Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

4
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File
 (NUREG - 0161)

5
 Exhibit 1 -
 Same Source

SUMMARY OF OPERATING EXPERIENCE

March 1999

The Indian Point Unit No. 3 Nuclear Power Plant was synchronized to the bus for a total of 683.68 hours, producing a gross generation of 620,190 MWe.

On March 9, at 1423 hours, a unit trip occurred following the installation of a temporary modification (TM) on one of three differential pressure flow transmitters (FT-436) associated with reactor coolant loop 3. Prior to installation of the TM, the low pressure supply side transmitter isolation valve was closed. After completion of TM installation, the transmitter supply side isolation valve was opened and a reactor trip occurred. Reopening the transmitter supply side isolation valve communicated a rapid pressure oscillation in the common high pressure side sensing line that was sufficient to actuate (trip) the bistable for flow transmitter FT-435. The bistable for FT-436 had previously been placed in trip and as a result of tripping a second bistable the reactor trip logic of two out of three low flow in a single loop was satisfied. The unit was stabilized in the hot shutdown condition.

During the period the plant was in the hot shutdown condition, the No. 31 and No. 32 main boiler feedwater pumps (MBFP) discharge check valve actuators (BFD-MOV-2-31, BFD-MOV-2-32), were discovered to be degraded and in need of repair. The reactor was taken critical on March 10, at 2302 hours, and the unit synchronized to the bus on March 12, at 0242 hours, using the No. 31 MBFP. Following successful repairs to BFD-MOV-2-32, No. 32 MBFP was returned to service and No. 31 MBFP was removed from service. Following successful repairs to BFD-MOV-2-31, No. 31 MBFP was returned to service. A plant load escalation commenced on March 17, at 2141 hours. The unit achieved full load on March 18, at approximately 1600 hours, and remained on line for the remainder of the reporting period.