

Indian Point 3  
Nuclear Power Plant  
P.O. Box 215  
Buchanan, New York 10511  
914 736.8001



**New York Power  
Authority**

**Robert J. Barrett**  
Site Executive Officer

January 14, 1998  
IPN-98-004

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

Subject: Indian Point 3 Nuclear Power Plant  
Docket No. 50-286  
License No. DPR-64  
**Monthly Operating Report for December 1997**

Dear Sir:

The attached monthly operating report, for the month of December 1997, is hereby submitted in accordance with Indian Point 3 Nuclear Power Plant Technical Specification 6.9.1.4.

The Authority is making no commitments in this letter.

Very truly yours,

A handwritten signature in black ink, appearing to be 'R. Barrett', written over a circular stamp.

Robert J. Barrett  
Site Executive Officer  
Indian Point 3 Nuclear Power Plant

Attachment

cc: See next page

9801260417 971231  
PDR ADOCK 05000286  
R PDR

2000003



cc: Mr. Hubert J. Miller  
Regional Administrator  
Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, Pennsylvania 19406-1415

Resident Inspector's Office  
Indian Point Unit 3  
U.S. Nuclear Regulatory Commission  
P.O. Box 337  
Buchanan, NY 10511

U.S. Nuclear Regulatory Commission  
ATTN: Director, Office of Information Resource Management  
Washington, D.C. 20555

INPO Records Center  
700 Galleria Parkway  
Atlanta, Georgia 30339-5957

# OPERATING DATA REPORT

DOCKET NO. 50-286  
 DATE 1-06-98  
 COMPLETED BY T. Orlando  
 TELEPHONE (914) 736-8340  
 IPN-98-004  
 ATTACHMENT I  
 PAGE 1 of 4

## OPERATING STATUS

1. Unit Name: Indian Point No. 3 Nuclear Power Plant
2. Reporting Period: December 1997
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report Give Reasons:

9. Power Level to Which Restricted, If Any (Net MWe): \_\_\_\_\_

10. Reasons for Restrictions, If Any: \_\_\_\_\_

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	8760	187,177
12. Number Of Hours Reactor Was Critical	422.47	4313.52	104,646.26
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	421.87	4653.21	102,204.64
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,266,274	13,445,523	290,377,707
17. Gross Electrical Energy Generated (MWH)	426,550	4,489,690	91,500,065
18. Net Electrical Energy Generated (MWH)	412,812	4,337,341	88,038,494
19. Unit Service Factor	56.7	53.1	54.6
20. Unit Availability Factor	56.7	53.1	54.6
21. Unit Capacity factor (Using MDC Net)	57.5	51.3	49.8*
22. Unit Capacity Factor (Using DER Net)	57.5	51.3	48.7
23. Unit Forced Outage Rate	43.3	11.4	29.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each): \_\_\_\_\_

25. If Shut Down At End Of Report Period. Estimated Date of Startup: January 2, 1998

26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

\* Weighted Average

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-286  
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 DATE 1-06-98  
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 PAGE 2 of 4

MONTH December 1997

DAY	AVERAGE DAILY POWER	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>982</u>	17	<u>982</u>
2	<u>981</u>	18	<u>523</u>
3	<u>981</u>	19	<u>0</u>
4	<u>981</u>	20	<u>0</u>
5	<u>981</u>	21	<u>0</u>
6	<u>980</u>	22	<u>0</u>
7	<u>980</u>	23	<u>0</u>
8	<u>980</u>	24	<u>0</u>
9	<u>981</u>	25	<u>0</u>
10	<u>982</u>	26	<u>0</u>
11	<u>981</u>	27	<u>0</u>
12	<u>982</u>	28	<u>0</u>
13	<u>981</u>	29	<u>0</u>
14	<u>981</u>	30	<u>0</u>
15	<u>982</u>	31	<u>0</u>
16	<u>982</u>		

## INSTRUCTIONS:

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-286  
UNIT NAME INDIAN POINT NO. 3  
DATE 1-6-98  
COMPLETED BY T. Orlando  
TELEPHONE (914) 736-8340  
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PAGE 3 of 4

REPORT MONTH December 1997

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR 3	LICENSEE EVENT REPORT #	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
8	971218	F	322.13	A	1	97-032-00	CF	CKTBRK A	Plant shutdown for an inoperable 480 volt Bus 6A due to a failure of the 32 Residual Heat Removal (RHR) pump circuit breaker to open following performance of surveillance test 3PT-M18, "RHR Pump Functional Test." 480 volt Bus 6A remained energized but was considered inoperable.

1  
F: Forced  
S: Scheduled

2  
Reason:  
A- Equipment  
B- Maintenance or Test  
C- Refueling  
D- Regulatory Restriction  
E- Operator Training & Licensee Examination  
F- Administrative  
G- Operational Error  
H- Other (Explain)

3  
Method:  
1-Manual  
2-Manual Scram  
3-Automatic Scram  
4-Other (Explain)

4  
Exhibit G - Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File  
(NUREG - 0161)

5  
Exhibit 1 -  
Same Source

## SUMMARY OF OPERATING EXPERIENCE

December 1997

The Indian Point Unit No. 3 Nuclear Power Plant was synchronized to the bus for a total of 421.87 hours producing a gross generation of 426,550 MWe.

On December 18, at approximately 1050 hours, the electrical circuit breaker for the 32 RHR pump failed to open following performance of surveillance test 3PT-M18, "RHR Functional Test." After several attempts to open the breaker, the electrical bus assigned to the breaker (480 volt Bus 6A) was declared inoperable and a unit shutdown commenced at approximately 1145 hours. The plant was shutdown because the Technical Specifications specify the requirement for four (4) 480 volt safety buses above cold shutdown which included bus 6A. With bus 6A declared inoperable the limiting condition for operation was not met and the Technical Specifications do not provide any action statement for that condition. The unit turbine was manually secured at 1352 hours, and the reactor was taken subcritical and hot shutdown achieved at 1428 hours. At approximately 1324 hours, the 32 RHR pump breaker opened without operator action.

During unit shutdown, with reactor power at approximately 71% power, control rod F02 dropped into the core, and a turbine runback occurred from approximately 70% to approximately 61% reactor power. The FSAR accident analysis section on dropped rod (section 14.1) states a turbine runback is prevented below 70% reactor power.

The unit was maintained in the hot shutdown condition while the investigation into the breaker failure progressed. The breaker is a 480 volt AC electrical circuit breaker, model DS-416 manufactured by Westinghouse. On December 22, the breaker for the 33 Pressurizer Backup Heater was determined to be potentially degraded when tested with its pole shaft reset spring removed. Engineering at that time could not provide a reasonable expectation of operability for DS-416 breakers used to power plant safety equipment. Without test results for the remaining breakers, Operations concluded there may be an unknown number of breakers that may be in a degraded condition and on December 23, at 0300 hours, declared the three Emergency Diesel Generators (EDGs) inoperable. Unit cooldown to cold shutdown commenced on December 23, at 0300 hours, and the plant entered cold shutdown on December 24, at 0240 hours. After further successful testing of 480 volt DS-416 breakers they were determined to be operable on December 26, at 1821 hours. Based on the operability determination for the breakers, the EDGs were declared operable on December 26, at 1925 hours. Plant personnel performed other maintenance and surveillance activities during the outage.

Following successful breaker extent of condition investigations, repairs, and testing the unit achieved hot shutdown on December 31, at 0457 hours, in preparation for plant re-start.

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Robert J. Barrett  
Site Executive Officer

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for Robert J. Barrett  
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Te24

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0.00007



980126091176PP

cc: Mr. Hubert J. Miller  
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475 Allendale Road  
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