Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 736.8001

New York Power Authority

Robert J. Barrett
Site Executive Officer

October 10, 1997
IPN-97-140

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
License No. DPR-64
Monthly Operating Report for September 1997

Dear Sir:

The attached monthly operating report, for the month of September 1997, is hereby submitted in accordance with Indian Point 3 Nuclear Power Plant Technical Specification 6.9.1.4.

The Authority is making no commitments in this letter.

Very truly yours,

FOR Robert J. Barrett
Site Executive Officer
Indian Point 3 Nuclear Power Plant

IE04

Attachment

cc: See next page

cc: Mr. Hubert J. Miller
Regional Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406-1415

Resident Inspector's Office
Indian Point Unit 3
U.S. Nuclear Regulatory Commission
P.O. Box 337
Buchanan, NY 10511

U.S. Nuclear Regulatory Commission
ATTN: Director, Office of Information Resource Management
Washington, D.C. 20555

INPO Records Center
700 Galleria Parkway
Atlanta, Georgia 30339-5957

OPERATING DATA REPORT

DOCKET NO. 50-286
DATE 10-1-97
COMPLETED BY T. Orlando
TELEPHONE (914) 736-8340
IPN-97-140
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OPERATING STATUS

1. Unit Name: Indian Point No. 3 Nuclear Power Plant
2. Reporting Period: September 1997
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report Give Reasons:

9. Power Level to Which Restricted, If Any (Net MWe):

10. Reasons for Restrictions, If Any:

		This Month	Yr-to-Date	Cumulative
11.	Hours In Reporting Period	720	6551	184,968
12.	Number Of Hours Reactor Was Critical	455.2	2926.05	102,758.79
13.	Reactor Reserve Shutdown Hours	0	0	0
14.	Hours Generator On-Line	326.89	2766.34	100,317.77
15.	Unit Reserve Shutdown Hours	0	0	0
16.	Gross Thermal Energy Generated (MWH)	750,456	7,922,969	284,855,153
17.	Gross Electrical Energy Generated (MWH)	229,770	2,634,700	89,645,075
18.	Net Electrical Energy Generated (MWH)	218,879	2,545,915	86,247,068
19.	Unit Service Factor	45.4	42.2	54.2
20.	Unit Availability Factor	45.4	42.2	54.2
21.	Unit Capacity factor (Using MDC Net)	31.5	40.3	49.3*
22.	Unit Capacity Factor (Using DER Net)	31.5	40.3	48.3
23.	Unit Forced Outage Rate	25.5	25.3	29.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each):

25. If Shut Down At End Of Report Period. Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

* Weighted Average

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-286
UNIT IP-3
DATE 10-1-97
COMPLETED BY T. Orlando
TELEPHONE (914) 736-8340
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MONTH September 1997

DAY	AVERAGE DAILY POWER	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	100
5	0	21	473
6	0	22	659
7	0	23	834
8	0	24	869
9	0	25	866
10	0	26	899
11	0	27	975
12	97	28	975
13	202	29	974
14	92	30	975
15	131	31	-
16	0		

INSTRUCTIONS:
On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-286
UNIT NAME INDIAN POINT NO. 3
DATE 10-1-97
COMPLETED BY T. Orlando.
TELEPHONE (914) 736-8340
IPN-97-140
ATTACHMENT I

REPORT MONTH September 1997

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR 3	LICENSEE EVENT REPORT #	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
3	970517	S	272.13	C	N/A	N/A	ZZ	ZZZZZZ	Scheduled cycle 9/10 refueling outage duration for the period.
4	970914	S	8.89	B	N/A	N/A	HA	TURBIN	Manually secured the turbine to facilitate the performance of surveillance test 3PT-V21, Turbine Generator Overspeed Trip Test.
5	970915	F	112.09	A	3	LER 97-023	IA	XXXXXX	Automatic reactor scram while performing surveillance test 3PT-Q95, Pressurizer Pressure Analog Functional Test.

1
F: Forced
S: Scheduled

2
Reason:
A- Equipment
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & Licensee Examination
F- Administrative
G- Operational Error
H- Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG - 0161)

5
Exhibit 1 - Same Source

SUMMARY OF OPERATING EXPERIENCE

September 1997

The Indian Point Unit No. 3 Nuclear Power Plant was synchronized to the bus for a total of 326.89 hours producing a gross generation of 229, 770 MWe.

On August 29, at 1343 hours, the unit entered the hot shutdown condition. On September 1, at 2230 hours, the unit achieved normal operating pressure and temperature.

On September 7, at 0129 hours, the reactor was taken critical. On September 10, at 0301 hours, prior to unit synchronization, the unit experienced a turbine trip and an automatic reactor scram. The trip occurred during performance of surveillance test 3PT-V06, "Turbine Generator Mechanical Trip Test." In accordance with the test procedure, operators closed the relay test switch stabs (H & I) going to the turbine auto stop trip solenoids (20/AST & 20/ASB) with the turbine generator lockout relays (86P/86BU) in the tripped condition. This operator action re-instated these trips into the turbine trip logic and initiated a turbine trip. A reactor trip occurred as designed because the permissive P-7 bypass of a turbine trip signal to the reactor protection system became defeated when prior to the test step its input from permissive P-10 was satisfied (two out of four power range neutron flux channels activated). The trip was caused by an error in the test procedure which did not require the turbine trip relays to be reset prior to re-instating the trips into the trip logic (see LER 97-024-00).

On September 11, at 0301 hours, the reactor was taken critical and the unit synchronized to the bus on September 12, at 0808 hours, ending the cycle 9/10 refueling outage.

On September 14, at 0320 hours, the unit was removed from service for a scheduled outage in order to perform surveillance test 3PT-V21, "Turbine Generator Overspeed Trip Test." Following successful completion of this test the unit was synchronized to the bus at approximately 1213 hours.

On September 15, at 1629 hours, the unit experienced an automatic reactor scram. The scram occurred during the performance of surveillance test 3PT-Q95, "Pressurizer Pressure Analog Functional Test." The trip occurred due to contacts (4-8) on Channel IV relay PC-474A-X-B not making up properly. When the test tripped the low pressurizer pressure reactor trip bistable for Channel I, the two out of four logic for a trip was made up. The plant was stabilized in the hot shutdown condition.

Following successful repairs, testing and an extent of condition investigation, the reactor was taken critical on September 19, at 1548 hours, and the unit synchronized to the bus on September 20, at 0834 hours.

Following various power level holds the unit achieved full power on September 27 and remained on line at full power for the remainder of the reporting period.