**Indian Point 3**
**Nuclear Power Plant**
P.O. Box 215
Buchanan, New York 10511
914 736.8001

**New York Power Authority**

**Robert J. Barrett**
Site Executive Officer

March 13, 1997
IPN-97-037

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
License No. DPR-64
**Monthly Operating Report for February 1997**

Dear Sir:

The attached monthly operating report, for the month of February 1997, is hereby submitted in accordance with Indian Point 3 Nuclear Power Plant Technical Specification 6.9.1.4.

The Authority is making no commitments in this letter.

Very truly yours,

For Robert J. Barrett
Site Executive Officer
Indian Point 3 Nuclear Power Plant

1/1
IE24

Attachment 210042

cc: See next page

cc: Hubert J. Miller
Regional Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406-1415

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
Indian Point 3 Nuclear Power Plant

John J. McOscar, Director
Division of Resource Management and Administration
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406-1415

INPO Records Center
700 Galleria Parkway
Atlanta, Georgia 30339-5957

OPERATING DATA REPORT

DOCKET NO. 50-286
DATE 3-3-97
COMPLETED BY T. Orlando
TELEPHONE (914) 736-8340
IPN-97-037
ATTACHMENT I
PAGE 1 of 4

OPERATING STATUS

1. Unit Name: Indian Point No. 3 Nuclear Power Plant
2. Reporting Period: February 1997
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report Give Reasons:

9. Power Level to Which Restricted, If Any (Net MWe):

10. Reasons for Restrictions, If Any:

| | | This Month | Yr-to-Date | Cumulative |
|---|---|---|---|---|
| 11. | Hours In Reporting Period | 672 | 1416 | 179,833 |
| 12. | Number Of Hours Reactor Was Critical | 248 | 678.5 | 101,011.24 |
| 13. | Reactor Reserve Shutdown Hours | 0 | 0 | 0 |
| 14. | Hours Generator On-Line | 217.22 | 647.1 | 98,198.53 |
| 15. | Unit Reserve Shutdown Hours | 0 | 0 | 0 |
| 16. | Gross Thermal Energy Generated (MWH) | 603,879 | 1,818,576 | 278,750,760 |
| 17. | Gross Electrical Energy Generated (MWH) | 202,060 | 607,310 | 87,617,685 |
| 18. | Net Electrical Energy Generated (MWH) | 195,203 | 586,554 | 84,287,707 |
| 19. | Unit Service Factor | 32.3 | 45.7 | 54.6 |
| 20. | Unit Availability Factor | 32.3 | 45.7 | 54.6 |
| 21. | Unit Capacity factor (Using MDC Net) | 30.1 | 42.9 | 49.6* |
| 22. | Unit Capacity Factor (Using DER Net) | 30.1 | 42.9 | 48.6 |
| 23. | Unit Forced Outage Rate | 67.7 | 54.3 | 30.2 |

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each): Cycle 9/10 Refueling Outage Scheduled to Commence May 1997
25. If Shut Down At End Of Report Period. Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation):

| | Forecast | Achieved |
|---|---|---|
| INITIAL CRITICALITY | | |
| INITIAL ELECTRICITY | | |
| COMMERCIAL OPERATION | | |

* Weighted Average

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-286
UNIT IP-3
DATE 3-3-97
COMPLETED BY T. Orlando
TELEPHONE (914) 736-8340
IPN-97-037
ATTACHMENT I
PAGE 2 of 4

MONTH February 1997

| DAY | AVERAGE DAILY POWER | DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) |
|---|---|---|---|
| 1 | 0 | 17 | 0 |
| 2 | 0 | 18 | 0 |
| 3 | 0 | 19 | 1 |
| 4 | 0 | 20 | 474 |
| 5 | 0 | 21 | 793 |
| 6 | 0 | 22 | 967 |
| 7 | 0 | 23 | 981 |
| 8 | 0 | 24 | 981 |
| 9 | 0 | 25 | 982 |
| 10 | 0 | 26 | 983 |
| 11 | 0 | 27 | 981 |
| 12 | 0 | 28 | 979 |
| 13 | 0 | 29 | - |
| 14 | 0 | 30 | - |
| 15 | 0 | 31 | - |
| 16 | 0 | | |

INSTRUCTIONS:
On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-286
UNIT NAME INDIAN POINT NO. 3
DATE 3-3-97
COMPLETED BY T. Orlando
TELEPHONE (914) 736-8340
IPN-97-037
ATTACHMENT I
PAGE 3 of 4

REPORT MONTH February 1997

| NO. | DATE | TYPE 1 | DURATION (HOURS) | REASON 2 | METHOD OF SHUTTING DOWN REACTOR 3 | LICENSEE EVENT REPORT # | SYSTEM CODE 4 | COMPONENT CODE 5 | CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE |
|---|---|---|---|---|---|---|---|---|---|
| 1 | 970118 | F | 314.17 | A | 1 | NA | CH | HTEXCH G | Removed the unit from service due to high level in No. 31C Feedwater Heater coincident with Feedwater Heaters 31/32B bypassed. |

1
F: Forced
S: Scheduled

2
Reason:
A- Equipment
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & Licensee Examination
F- Administrative
G- Operational Error
H- Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG - 0161)

5
Exhibit 1 - Same Source

# SUMMARY OF OPERATING EXPERIENCE

February 1997

The Indian Point Unit No. 3 Nuclear Power Plant was synchronized to the bus for a total of 217.22 hours, producing a gross generation of 202,060 MWe.

On January 18, at 1635 hours, a load reduction commenced in preparation to remove the plant from service. The turbine was manually shutdown at 2150 hours, and the reactor manually shutdown at 2230 hours. Plant shutdown was necessary due to a high level in Feedwater Heater No. 31C coincident with Feedwater Heaters No. 31B/32B bypassed due to leaks.

While the plant was in the hot shutdown condition, a leak was observed on the pressurizer manway. A decision was made to bring the unit to the cold shutdown condition to make the necessary repairs. On January 21, at 1528 hours, hot shutdown was exited to bring the unit to cold shutdown. Cold shutdown was achieved at 1442 hours on January 23. The outage was extended in order to perform various refueling and surveillance tests, and replace two (2) pressurizer power operated relief valves (PORVs).

On February 15, at 1123 hours, the unit reached the hot shutdown condition. On February 18, at 1600 hours, the reactor was brought critical and the unit was synchronized to the bus on February 19, at 2247 hours. The unit achieved full power on February 23, at 0130 hours, and remained on line at full power for the remainder of the reporting period.