

Indian Point 3  
Nuclear Power Plant  
P.O. Box 215  
Buchanan, New York 13511  
914 736 8007



Robert J. Barrett  
Site Executive Officer

November 21, 1997  
IPN-97-159

U.S. Nuclear Regulatory Commission  
ATTN.: Document Control Desk  
Washington, D.C. 20555

SUBJECT: Indian Point 3 Nuclear Power Plant  
Docket No. 50-286  
License No. DPR-64  
**Licensee Event Report # 97-028-00,**  
**"The Safety Injection System Was Aligned for Testing Contrary to**  
**The Technical Specifications Due to Personnel Errors"**

Dear Sir:

The attached Licensee Event Report (LER) 97-028-00 is hereby submitted as required by 10 CFR 50.73. This event is of the type defined in 10 CFR 50.73 (a)(2)(i)(B).

Also attached are the commitments made by the Authority in this LER.

Very truly yours,

A handwritten signature in cursive script that reads 'Robert J. Barrett'.

Robert J. Barrett  
Site Executive Officer  
Indian Point 3 Nuclear Power Plant

Attachments

cc: See next page

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PDR ADOCK 05000286  
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cc: Mr. Hubert J. Miller  
Regional Administrator  
Region I  
U. S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, Pennsylvania 19406-1415

INPO Record Center  
700 Galleria Parkway  
Atlanta, Georgia 30339-5957

U.S. Nuclear Regulatory Commission  
Resident Inspectors' Office  
Indian Point 3 Nuclear Power Plant

COMMITMENT LIST

Number	Commitment	Due
IPN-97-159-01	Operations will revise system operating procedures (SOP-SI-1 and SOP-RHR-1) to provide a precaution and limitation for the system valves that must not be energized or repositioned when the reactor coolant system is above 350 degrees F. The revision will provide a precaution and limitation for not entering two safety injection LCO action statements simultaneously.	12/05/97
IPN-97-159-02	Performance engineering will provide an appropriate test method for future tests of the safety injection valves that must not be energized or repositioned when reactor coolant system is above 350 degrees F. This type of valve is tested during startup from a major outage and the next major outage is scheduled for 1999.	03/31/98
IPN-97-159-03	Training will provide this event as a lesson learned during a licensed operator requalification training course.	04/30/98
IPN-97-159-04	Training will add the basis for the technical specification regarding safety injection valve positions to the initial licensed operator (ILO) training program. They will also add to the ILO training program the prohibition on energizing these valves or changing their position with the reactor coolant system above 350 degrees F.	04/30/98