

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 736.8001



Robert J. Barrett
Site Executive Officer

September 25, 1997
IPN-97-131

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555


SUBJECT: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
License No. DPR-64
Licensee Event Report # 97-019-00
**Steam Driven Auxiliary Feedwater Pump Full Flow Testing Inadequate
Due To Improper Procedure Change In 1978, A Condition Prohibited
By Technical Specification**

Dear Sir:

The attached Licensee Event Report (LER) 97-019-00 is hereby submitted as required by 10 CFR 50.73. This event is of the type defined in 10 CFR 50.73 (a)(2)(i)(B).

Also attached is the commitment made by the Authority in this LER.

Very truly yours,


Robert J. Barrett
Site Executive Officer
Indian Point 3 Nuclear Power Plant

Attachments

cc: See next page



9710070102 970925
PDR ADOCK 05000286
S PDR

cc: Hubert J. Miller
Regional Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406-1415

INPO Record Center
700 Galleria Parkway
Atlanta, Georgia 30339-5957

U.S. Nuclear Regulatory Commission
Resident Inspectors' Office
Indian Point 3 Nuclear Power Plant

COMMITMENT LIST

Number	Commitment	Due
IPN-97-131-01	Engineering will continue to work with Westinghouse to provide an analysis using an approved computer code for Loss of Normal Feedwater, which bounds the Loss of Offsite Power accident scenario, allowing for the effect, capacity and functionality of the steam driven auxiliary feedwater pump, including maximum allowable delay time prior to pump full flow. Engineering will perform a safety evaluation based on the analysis and revise the FSAR where appropriate.	March 31, 1998.