Indian Point 3 Nuclear Power Plant P.O. Box 215 Buchanan, New York 10511 914 736.8001



Robert J. Barrett Site Executive Officer

September 25, 1997 IPN-97-131

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

SUBJECT: Indian Point 3 Nuclear Power Plant Docket No. 50-286 License No. DPR-64 Licensee Event Report # 97-019-00 Steam Driven Auxiliary Feedwater Pump Full Flow Testing Inadequate Due To Improper Procedure Change In 1978, A Condition Prohibited By Technical Specification

Dear Sir:

The attached Licensee Event Report (LER) 97-019-00 is hereby submitted as required by 10 CFR 50.73. This event is of the type defined in 10 CFR 50.73 (a)(2)(i)(B).

Also attached is the commitment made by the Authority in this LER.

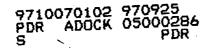
Very truly yours,

Robert J. Barrett Site Executive Officer Indian Point 3 Nuclear Power Plant

Attachments

cc: See next page

t. C.





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Hubert J. Miller Regional Administrator Region I U.S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, Pennsylvania 19406-1415

INPO Record Center 700 Galleria Parkway Atlanta, Georgia 30339-5957

U.S. Nuclear Regulatory Commission Resident Inspectors' Office Indian Point 3 Nuclear Power Plant



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COMMITMENT LIST

Number	Commitment	Due
IPN-97-131-01	Engineering will continue to work with Westinghouse to provide an analysis using an approved computer code for Loss of Normal Feedwater, which bounds the Loss of Offsite Power accident scenario, allowing for the effect, capacity and functionality of the steam driven auxiliary feedwater pump, including maximum allowable delay time prior to pump full flow. Engineering will perform a safety evaluation based on the analysis and revise the FSAR where appropriate.	March 31, 1998.