Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 736.8001

New York Power Authority

Robert J. Barrett
Plant Manager

September 11,1996
IPN-96-102

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
License No. DPR-64
Monthly Operating Report for August 1996

Dear Sir:

The attached monthly operating report, for the month of August 1996, is hereby submitted in accordance with Indian Point 3 Nuclear Power Plant Technical Specification 6.9.1.4.

The Authority is making no commitments in this letter.

Very truly yours,

Robert J. Barrett
Plant Manager
Indian Point 3 Nuclear Power Plant

Attachment

cc: See next page

IE24

cc: Hubert J. Miller
Regional Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406-1415

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
Indian Point 3 Nuclear Power Plant

John J. McOscar, Director
Division of Resource Management and Administration
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406-1415

INPO Records Center
700 Galleria Parkway
Atlanta, Georgia 30339-5957

OPERATING DATA REPORT

DOCKET NO. 50-286
DATE 9-4-96
COMPLETED BY T. Orlando
TELEPHONE (914) 736-8340
IPN-96-102
ATTACHMENT I
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OPERATING STATUS

1. Unit Name: Indian Point No. 3 Nuclear Power Plant
2. Reporting Period: August 1996
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report Give Reasons:

9. Power Level to Which Restricted, If Any (Net MWe):

10. Reasons for Restrictions, If Any:

		This Month	Yr-to-Date	Cumulative
11.	Hours In Reporting Period	744	5855	175,488
12.	Number Of Hours Reactor Was Critical	744	3,640.21	97,403.74
13.	Reactor Reserve Shutdown Hours	0	0	0
14.	Hours Generator On-Line	744	3,474.18	94,635.01
15.	Unit Reserve Shutdown Hours	0	0	0
16.	Gross Thermal Energy Generated (MWH)	2,185,779	10,037,481	268,766,362
17.	Gross Electrical Energy Generated (MWH)	728,280	3,351,620	84,271,525
18.	Net Electrical Energy Generated (MWH)	701,546	3,231,738	81,060,401
19.	Unit Service Factor	100	59.3	53.9
20.	Unit Availability Factor	100	59.3	53.9
21.	Unit Capacity factor (Using MDC Net)	97.7	57.2	48.9*
22.	Unit Capacity Factor (Using DER Net)	97.7	57.2	47.9
23.	Unit Forced Outage Rate	0	40.7	30.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each):

25. If Shut Down At End Of Report Period. Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

* Weighted Average

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-286
UNIT IP-3
DATE 9-4-96
COMPLETED BY T. Orlando
TELEPHONE (914) 736-8340
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MONTH August 1996

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	971	17	968
2	970	18	967
3	971	19	967
4	969	20	967
5	970	21	967
6	970	22	967
7	969	23	954
8	968	24	958
9	967	25	612
10	966	26	657
11	967	27	956
12	967	28	963
13	968	29	965
14	937	30	965
15	938	31	965
16	967		

INSTRUCTIONS:
On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-286
UNIT NAME INDIAN POINT NO. 3
DATE 9-4-96
COMPLETED BY T. Orlando
TELEPHONE (914) 736-8340
IPN-96-102
ATTACHMENT I

REPORT MONTH August 1996

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR 3	LICENSEE EVENT REPORT #	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
7	960814	F	NA	B	NA	NA	EG	ELECON	AUTOMATIC TURBINE RUNBACK TO APPROXIMATELY 80% REACTOR POWER DUE TO A VOLTAGE INTERRUPTION (SPIKE) ON NO. 34 INSTRUMENT BUS.
8	960825	F	NA	A	NA	NA	CJ	VALVEX F	REDUCED LOAD TO APPROXIMATELY 60% REACTOR POWER TO DECREASE RADIATION FIELDS TO FACILITATE MAINTENANCE ON VALVE STEM SEAL ON CONTAINMENT SPRAY VALVE RC-PCV-455B.

1
F: Forced
S: Scheduled

2
Reason:
A- Equipment
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & Licensee Examination
F- Administrative
G- Operational Error
H- Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG - 0161)

5
Exhibit 1 - Same Source

SUMMARY OF OPERATING EXPERIENCE

AUGUST 1996

The Indian Point Unit No. 3 Nuclear Power Plant was synchronized to the bus for a total of 744 hours producing a gross generation of 728,280 MWE.

On August 14, at approximately 2020 hours, a turbine runback to approximately 80% reactor power occurred. The turbine runback occurred while plant personnel were reinstalling instrument power fuses for radiation monitor R-13. The installation of the fuses caused a voltage interruption ("Spike") on No. 34 instrument bus. The voltage spike on instrument bus voltage caused 1/4 Nuclear Power Range Indication Channels (NI-43) to actuate logic for a NIS Power Range Dropped Rod/Rod Stop resulting in automatic initiation of a turbine load runback. Plant load was stabilized at approximately 800 MWE. Following investigation and necessary repairs a load increase commenced on August 15, at 0230 hours. Full power was achieved at 1000 hours.

On August 24, at 1825 hours, plant operators observed that the Reactor Coolant Drain Tank (RCDT) high temperature alarm was annunciating and that tank temperature and pressure were increasing. Operators concluded that the increasing temperature and pressure in the RCDT was a result of a primary system leak into the tank. Operator investigation determined that the leakage into the RCDT was from the valve leak-off collection system. On August 25, at 0100 hours, a load reduction commenced in order to facilitate investigation and repairs of the leakage. At 0453 hours the unit load decrease was stopped at approximately 62% reactor power. Investigation into the source of leakage determined that the bellows seal for the valve stem of containment spray valve RC-PCV-455B failed. The leak-off for the valve stem bellow seal was isolated at 0515 hours stopping leak-off input to the RCDT. Following adjustment of the valve stem packing seal, a load increase commenced on August 26, at 1400 hours and full load was achieved on August 27, at approximately 0600 hours.

The unit remained on line at or near full power for the remainder of the reporting period.