

**Nuclear Power Plant**  
P.O. Box 215  
Buchanan, New York 10511  
914 736.8001



**Robert J. Barrett**  
Plant Manager

May. 14, 1996  
IPN-96-060

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

Subject: Indian Point 3 Nuclear Power Plant  
Docket No. 50-286  
License No. DPR-64  
**Monthly Operating Report for April 1996**

Dear Sir:

The attached monthly operating report, for the month of April 1996, is hereby submitted in accordance with Indian Point 3 Nuclear Power Plant Technical Specification 6.9.1.4.

The Authority is making no commitments in this letter.

Very truly yours,

  
Robert J. Barrett  
Plant Manager  
Indian Point 3 Nuclear Power Plant

Attachment

cc: See next page

9605200266 960430  
PDR ADOCK 05000286  
R PDR

200138

JE24.1

cc: Thomas T. Martin  
Regional Administrator  
Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, Pennsylvania 19406-1415

U.S. Nuclear Regulatory Commission  
Resident Inspectors' Office  
Indian Point 3 Nuclear Power Plant

John J. McOscar, Director  
Division of Resource Management and Administration  
Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, Pennsylvania 19406-1415

INPO Records Center  
700 Galleria Parkway  
Atlanta, Georgia 30339-5957

# OPERATING DATA REPORT

DOCKET NO. 50-286  
 DATE 5-1-96  
 COMPLETED BY T. Orlando  
 TELEPHONE (914) 736-8340  
 IPN-96-060  
 ATTACHMENT I  
 PAGE 1 of 4

## OPERATING STATUS

1. Unit Name: Indian Point No. 3 Nuclear Power Plant
2. Reporting Period: April 1996
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report Give Reasons:

9. Power Level to Which Restricted, If Any (Net MWe):

10. Reasons for Restrictions, If Any:

|   | This Month | Yr-to-Date | Cumulative  |
|---|------------|------------|-------------|
| 11. Hours In Reporting Period               | 719        | 2903       | 172,536     |
| 12. Number Of Hours Reactor Was Critical    | 654.92     | 723.25     | 94,486.78   |
| 13. Reactor Reserve Shutdown Hours          | 0          | 0          | 0           |
| 14. Hours Generator On-Line                 | 582.18     | 582.18     | 91,743.01   |
| 15. Unit Reserve Shutdown Hours             | 0          | 0          | 0           |
| 16. Gross Thermal Energy Generated (MWH)    | 1,618,000  | 1,619,042  | 260,347,923 |
| 17. Gross Electrical Energy Generated (MWH) | 539,240    | 539,240    | 81,459,145  |
| 18. Net Electrical Energy Generated (MWH)   | 520,474    | 520,474    | 78,349,137  |
| 19. Unit Service Factor                     | 81.0       | 20.1       | 53.2        |
| 20. Unit Availability Factor                | 81.0       | 20.1       | 53.2        |
| 21. Unit Capacity factor (Using MDC Net)    | 75.0       | 18.6       | 48.2*       |
| 22. Unit Capacity Factor (Using DER Net)    | 75.0       | 18.6       | 47.1        |
| 23. Unit Forced Outage Rate                 | 19.0       | 79.9       | 31.2        |

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each):

25. If Shut Down At End Of Report Period. Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

|                      | Forecast | Achieved |
|----------------------|----------|----------|
| INITIAL CRITICALITY  |          |          |
| INITIAL ELECTRICITY  |          |          |
| COMMERCIAL OPERATION |          |          |

\* Weighted Average

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-286  
 UNIT IP-3  
 DATE 5-1-96  
 COMPLETED BY T. Orlando  
 TELEPHONE (914) 736-8340  
 IPN-96-060  
 ATTACHMENT I  
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MONTH APRIL, 1996

| DAY | AVERAGE DAILY POWER<br>LEVEL (MWe-Net) | DAY | AVERAGE DAILY POWER LEVEL<br>(MWe-Net) |
|-----|--|-----|--|
| 1   | <u>0</u>                               | 17  | <u>968</u>                             |
| 2   | <u>1</u>                               | 18  | <u>968</u>                             |
| 3   | <u>0</u>                               | 19  | <u>967</u>                             |
| 4   | <u>0</u>                               | 20  | <u>968</u>                             |
| 5   | <u>0</u>                               | 21  | <u>967</u>                             |
| 6   | <u>13</u>                              | 22  | <u>967</u>                             |
| 7   | <u>202</u>                             | 23  | <u>967</u>                             |
| 8   | <u>455</u>                             | 24  | <u>968</u>                             |
| 9   | <u>770</u>                             | 25  | <u>969</u>                             |
| 10  | <u>897</u>                             | 26  | <u>967</u>                             |
| 11  | <u>948</u>                             | 27  | <u>968</u>                             |
| 12  | <u>954</u>                             | 28  | <u>970</u>                             |
| 13  | <u>974</u>                             | 29  | <u>970</u>                             |
| 14  | <u>975</u>                             | 30  | <u>970</u>                             |
| 15  | <u>975</u>                             | 31  | <u>-----</u>                           |
| 16  | <u>970</u>                             |     |  |

## INSTRUCTIONS:

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-286  
 UNIT NAME INDIAN POINT NO. 3  
 DATE 5-1-96  
 COMPLETED BY T. Orlando  
 TELEPHONE (914) 736-8340  
 IPN-96-060  
 ATTACHMENT I  
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REPORT MONTH APRIL 1996

| NO. | DATE   | TYPE<br>1 | DURATION<br>(HOURS) | REASON<br>2 | METHOD OF<br>SHUTTING<br>DOWN<br>REACTOR 3 | LICENSEE EVENT<br>REPORT # | SYSTEM<br>CODE<br>4 | COMPONENT<br>CODE<br>5 | CAUSE & CORRECTIVE ACTION TO<br>PREVENT RECURRENCE  |
|-----|--------|-----------|---------------------|-------------|--|----------------------------|---------------------|------------------------|---|
| 1   | 950914 | F         | 38.92               | A           | 1  | 95-018-00                  | XX                  | GENERA X               | DURING A CONTROLLED UNIT SHUTDOWN FOR REPAIR OF A HYDROGEN LEAK IN THE GENERATOR HYDROGEN COOLER THE REACTOR WAS MANUALLY TRIPPED AND THE TURBINE AUTOMATICALLY TRIPPED DUE TO AN ALARM FOR A HIGH MAIN GENERATOR STATOR DIFFERENTIAL TEMPERATURE DUE TO STATOR COOLER GAS BINDING. THE OUTAGE WAS EXTENDED TO FACILITATE REPAIRS TO OTHER PLANT SYSTEMS. |
| 2   | 960402 | F         | 97.90               | F           | 2  | 96-009-00                  | XX                  | GENERA D               | MANUALLY TRIPPED THE REACTOR AND THE TURBINE AUTOMATICALLY TRIPPED IN RESPONSE TO AN ALARM FOR A HIGH DIFFERENTIAL TEMPERATURE IN THE MAIN GENERATOR STATOR. THE ALARM WAS CAUSED BY REDUCED COOLING FLOW IN THE STATOR COOLERS DUE TO INADEQUATE VENTING AND FLOW BALANCING.   |

1  
F: Forced  
S: Scheduled

2  
Reason:  
A- Equipment  
B- Maintenance or Test  
C- Refueling  
D- Regulatory Restriction  
E- Operator Training & Licensee Examination  
F- Administrative  
G- Operational Error  
H- Other (Explain)

3  
Method:  
1-Manual  
2-Manual Scram  
3-Automatic Scram  
4-Other (Explain)

4  
Exhibit G - Instructions  
for Preparation of Data  
Entry Sheets for Licensee  
Event Report (LER) File  
(NUREG - 0161)

5  
Exhibit 1 -  
Same Source

## SUMMARY OF OPERATING EXPERIENCE

APRIL 1996

The Indian Point Unit No. 3 Nuclear Power Plant was synchronized to the bus for a total of 582.18 hours producing a gross generation of 539,240 MWE. The reactor had been brought critical on March 29, at 0340 hours and the unit was synchronized to the bus on April 2, at 1455 hours. At approximately 1612 hours, when power was approximately 13 percent, operators initiated a manual reactor trip (and a turbine trip occurred automatically) in response to an alarm for a high differential temperature on the unit's main generator stator. The stator high differential temperature was due to a reduction of service water flow to the 33 hydrogen cooler as a result of gas binding and inadequate cooling water flow balance between the four hydrogen coolers of the main generator stator. The event was reported in LER 96-009. After revision of plant procedures, the reactor was brought critical on April 5, at 0817 hours. The hydrogen coolers were vented and flow balanced, the unit was synchronized to the bus on April 6, at 1806 hours and a controlled load increase ensued. The unit achieved 99 percent reactor power on April 12, at 1915 hours, and remained on line at approximately 99 percent reactor power for the remainder of the reporting period. The unit was administratively controlled at approximately 99 percent power while troubleshooting the automatic control system for boric acid makeup.