

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 736.8001



L. M. Hill
Site Executive Officer

February 14, 1996
IPN-96-013

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555


Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
License No. DPR-64
Monthly Operating Report for January 1996

Dear Sir:

The attached monthly operating report, for the month of January 1996, is hereby submitted in accordance with Indian Point 3 Nuclear Power Plant Technical Specification 6.9.1.4.

The Authority is making no commitments in this letter.

Very truly yours,


L.M. Hill
Site Executive Officer
Indian Point 3 Nuclear Power Plant

LMH/cl

Attachment

cc: See next page

9602220143 960131
PDR ADDCK 05000286
R PDR

IE24
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cc: Thomas T. Martin
Regional Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406-1415

U.S. Nuclear Regulatory Commission
Resident Inspectors' Office
Indian Point 3 Nuclear Power Plant

John J. McOscar, Director
Division of Resource Management and Administration
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406-1415

INPO Records Center
700 Galleria Parkway
Atlanta, Georgia 30339-5957

OPERATING DATA REPORT

DOCKET NO. 50-286
 DATE 2-1-96
 COMPLETED BY T. Orlando
 TELEPHONE (914) 736-8340
 IPN-96-013
 ATTACHMENT I
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OPERATING STATUS

1. Unit Name: Indian Point No. 3 Nuclear Power Plant
2. Reporting Period: January 1996
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report Give Reasons: _____
9. Power Level to Which Restricted, If Any (Net MWe): _____
10. Reasons for Restrictions, If Any: _____

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	744	170,377
12. Number Of Hours Reactor Was Critical	0	0	93,763.53
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	0	0	91,160.83
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	0	0	258,728,881
17. Gross Electrical Energy Generated (MWH)	0	0	80,919,905
18. Net Electrical Energy Generated (MWH)	0	0	77,828,663
19. Unit Service Factor	0	0	53.5
20. Unit Availability Factor	0	0	53.5
21. Unit Capacity factor (Using MDC Net)	0	0	48.5*
22. Unit Capacity Factor (Using DER Net)	0	0	47.3
23. Unit Forced Outage Rate	100	100	30.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each): _____

25. If Shut Down At End Of Report Period. Estimated Date of Startup: February 26, 1996
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

* Weighted Average

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-286
UNIT	IP-3
DATE	2-1-96
COMPLETED BY	T. Orlando
TELEPHONE	(914) 736-8340
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MONTH JANUARY, 1996

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

INSTRUCTIONS:

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-286
 UNIT NAME INDIAN POINT NO. 3
 DATE 2-1-96
 COMPLETED BY T. Orlando
 TELEPHONE (914) 736-8340
 IPN-96-013
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REPORT MONTH JANUARY 1996

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR 3	LICENSEE EVENT REPORT #	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
1	950914	F	744	A	1	95-18-00	XX	GENERA X	THE REACTOR WAS MANUALLY SHUTDOWN AND THE TURBINE AUTOMATICALLY SHUTDOWN DUE TO A HIGH MAIN GENERATOR STATOR TEMPERATURE DIFFERENTIAL (DELTA T) DURING A CONTROLLED UNIT SHUTDOWN. THIS SHUTDOWN WAS REQUIRED TO REPAIR A HYDROGEN LEAK IN THE UNITS MAIN GENERATOR. THE OUTAGE WAS EXTENDED TO FACILITATE REPAIRS TO OTHER PLANT SYSTEMS.

1
 F: Forced
 S: Scheduled

2
 Reason:
 A-Equipment
 B-Maintenance or Test
 C-Refueling
 D- Regulatory Restriction

3
 Method
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

4
 Exhibit - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File
 (NUREG - 0161)

5
 Exhibit - Same Source

SUMMARY OF OPERATING EXPERIENCE

JANUARY 1996

The plant was in cold shutdown as a result of an extended forced outage from a technical specification required shutdown for a leak of a service water system containment isolation valve that occurred on October 21, 1995. The forced outage was extended in order to facilitate repairs to other plant systems and to perform an extensive review of operational procedures, including procedure revisions and upgrades. On December 2, 1995, in preparation for plant startup, the unit exited cold shutdown and entered hot standby on December 14, 1995, and achieved normal reactor coolant temperature and pressure on December 17, 1995. The plant remained in hot standby until December 22, 1995, when a handhole leak was discovered on a steam generator. To repair the leak a unit cooldown was initiated on December 23, 1995. Cold shutdown was achieved on December 25, 1995, at 2015 hours.

While in cold shutdown on January 20, 1996, at approximately 1845 hours, the unit experienced a loss of 138 Kv supply power to the Station Auxiliary Transformer (LOOP). A preliminary assessment of the event determined that all three Emergency Diesel Generators (EDG) automatically started as required by sensing the undervoltage conditions on the 480 volt safeguards buses. However, shortly after starting, the output breaker for loading EDG 31 to its assigned 480 volt safeguard bus tripped opened rendering EDG 31 inoperable. EDG 32 and EDG 33 successfully started and their associated output breakers closed in and powered their assigned 480 volt safeguard buses. However, after offsite power was restored EDG 32 was declared inoperable at approximately 2229 hours, when an operator discovered the EDG 32 room ventilation was not operating as required. Declaring EDG 32 inoperable along with an inoperable EDG 31 placed the plant in violation of Technical Specifications Section 3.7.F.4 which requires two EDGs to be operable in cold shutdown. Upon investigation it was determined that there was insufficient air pressure to the room's ventilation system to operate as required. The pressure regulator for the ventilation system was replaced. The ventilation system was tested and placed in service restoring EDG 32 to operable.

Preliminary investigation of the event discovered that the cause of the LOOP was a faulted phase gap-type surge arrester on Phase A of the Station Auxiliary Transformer. The arrester faulted as a result of moisture intrusion through a caulked seal in the arrester's upper cover. Preliminary results of troubleshooting and testing of EDG-31 and its output breaker indicated that the failure of the breaker to close-in was caused by a loose wire termination in a seal-in contact relay in the EDG undervoltage circuit. A preliminary failure investigation of the air regulator determined that rust was the cause of the improper operation. Due to these events and the need to make repairs and retest plant equipment, the unit remained off line and in the cold shutdown condition for the entire reporting period.