

Omaha Public Power District
444 South 16th Street Mall
Omaha, Nebraska 68102-2247
402/636-2000

November 15, 1995
LIC-95-0212

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, D.C. 20555

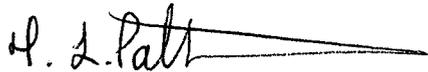
Reference: Docket No. 50-285

SUBJECT: October 1995 Monthly Operating Report (MOR)

Enclosed please find the October 1995 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification 5.9.1.

If you should have any questions, please contact me.

Sincerely,



T. L. Patterson
Division Manager
Nuclear Operations

TLP/d11

Enclosures

c: Winston & Strawn
L. J. Callan, NRC Regional Administrator, Region IV
L. R. Wharton, NRC Project Manager
W. C. Walker, NRC Senior Resident Inspector
R. T. Pearce, Combustion Engineering
R. J. Simon, Westinghouse
INPO Records Center

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PDR ADOCK 05000286
R PDR



OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

OCTOBER 1995
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of October, Fort Calhoun Station (FCS) operated at a nominal 100% power. Normal plant maintenance, surveillance, equipment rotation activities and scheduled on-line modifications occurred during the month. Monitoring of a minor Control Element Drive Mechanism (CEDM) mechanical seal leak continued.

An Emergency Plan drill was conducted on October 3rd. An inadvertent activation of the Emergency Notification System (ENS) sirens in Harrison County, Iowa, occurred during this drill requiring a 4-hour notification to the NRC.

On October 12th, two Omaha Public Power District employees received acid burns during scheduled maintenance in the Water Plant when a badly corroded line parted. One individual received severe burns to the neck and upper chest, while the other received only minor burns to the arm. Both individuals were transported to off-site facilities for medical attention. The State of Nebraska Department of Environmental Quality was notified of the spill and a 4-hour report was made to the NRC.

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of October, no Power Operated Relief Valve (PORV) or primary system safety valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

Although above normal, the October Reactor Coolant System (RCS) leak rate was steady at approximately 0.4 to 0.5 gpm throughout the month. This leak rate remained relatively steady following the reactor trip and resultant surveillance testing of the CEDMs on August 26, 1995.

The major contributor to the increase in RCS leakage has been classified as "Known" leakage. This leakage is being collected in the Reactor Coolant Drain Tank (RCDT). The leakage source has been attributed to seal leakage from CEDM #15. The "Known" leak rate has stabilized at 0.26 gpm. The remainder of the leakage has been classified as "Unknown" leakage.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

<u>Amendment No.</u>	<u>Description</u>
None	

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF OCTOBER 1995

- Replaced Raw Water Pump AC-10A due to normal wear
- Rebuilt Charging Pump CH-1C due to excessive packing leaks
- Replaced the control switch for YCV-1045, the steam inlet valve to Auxiliary Feed Pump FW-10
- Installed a new Containment Isolation Actuation Signal (CIAS) lockout relay 86B\CIAS that failed to trip during a regularly scheduled surveillance

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	NOVEMBER 03, 1995
COMPLETED BY	D. L. LIPPY
TELEPHONE	(402) 533-6843

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
 2. Reporting Period: OCTOBER 1995

NOTES

3. Licensed Thermal Power (MWt): 1500
 4. Nameplate Rating (Gross MWe): 502
 5. Design Elec. Rating (Net MWe): 478
 6. Max. Dep. Capacity (Gross MWe): 502
 7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	745.0	7296.0	193730.0
12. Number of Hours Reactor was Critical	745.0	5826.1	152244.0
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	745.0	5742.2	150516.5
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1115149.8	8347260.6	200495961.1
17. Gross Elec. Energy Generated (MWH)..	377414.0	2782606.0	66187488.2
18. Net Elec. Energy Generated (MWH)....	360496.4	2651999.8	63143792.1
19. Unit Service Factor.....	100.0	78.7	77.7
20. Unit Availability Factor.....	100.0	78.7	77.7
21. Unit Capacity Factor (using MDC Net)	101.2	76.0	70.5
22. Unit Capacity Factor (using DER Net)	101.2	76.0	68.9
23. Unit Forced Outage Rate.....	.0	4.6	4.0

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
A MAINTENANCE OUTAGE IS SCHEDULED TO OCCUR FROM MARCH 16-23, 1996 TO REPAIR/REPLACE DEGRADING CEDM MECHANICAL SEALS.

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.): Forecast Achieved

INITIAL CRITICALITY		_____	_____
INITIAL ELECTRICITY	N/A	_____	_____
COMMERCIAL OPERATION		_____	_____

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE NOVEMBER 03, 1995
COMPLETED BY D. L. LIPPY
TELEPHONE (402) 533-6843

MONTH OCTOBER 1995

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	480	17	484
2	480	18	484
3	480	19	480
4	481	20	482
5	482	21	485
6	483	22	486
7	484	23	486
8	484	24	486
9	484	25	486
10	484	26	486
11	484	27	486
12	483	28	486
13	482	29	486
14	483	30	487
15	484	31	487
16	484		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
 UNIT NAME Fort Calhoun St.
 DATE November 3, 1995
 COMPLETED BY D. L. Lippy
 TELEPHONE (402) 533-6843

REPORT MONTH October 1995

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
None.									

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit H - Same Source

Attachment IV
Refueling Information
Fort Calhoun Station - Unit No. 1

Report for the month ending October 31, 1995

1. Scheduled date for next refueling shutdown. September 21, 1996
2. Scheduled date for restart following refueling. November 2, 1996
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? Yes
 - a. If answer is yes, what, in general, will these be? Enrichment limit of spent fuel racks is to be increased to at least 4.5 w/o from 4.2 w/o. This is necessary based upon the preliminary Cycle 17 core pattern development.
 - b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. N/A
 - c. If no such review has taken place, when is it scheduled? N/A
4. Scheduled date(s) for submitting proposed licensing action and support information. January 1996 (for spent fuel rack enrichment limit change)
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. N/A
6. The number of fuel assemblies:
 - a) in the core 133 Assemblies
 - b) in the spent fuel pool 618 Assemblies
 - c) spent fuel pool storage capacity 1083 Assemblies
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 2007 Outage

Prepared by *Kevin Shultz*

Date 11-6-95