Indian Point 3 Nuclear Power Plant P.O. Box 215 Buchanan, New York 10511 914 736.8001



L. M. Hill Site Executive Officer

August: 11,1995 IPN-95-086

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Subject: Indian Point 3 Nuclear Power Plant

Docket No. 50-286 License No. DPR-64

Monthly Operating Report for July 1995

Dear Sir:

The attached monthly operating report, for the month of July 1995, is hereby submitted in accordance with Indian Point 3 Nuclear Power Plant Technical Specification 6.9.1.4. During the operating period the pressurizer power operated relief valves (PORVs) cycled. In accordance with Technical Specification 6.9.1.4 a challenge to the PORVs is being documented by this report.

The Authority is making no commitments in this letter.

Very truly yours,

//M Hill

Site Executive Officer

Indian Point 3 Nuclear Power Plant

LMH/cbr

Attachment

cc: See next page

9508220294 950731 PDR ADDCK 05000286 R PDR JEZY.

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cc: Thomas T. Martin
Regional Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406-1415

U.S. Nuclear Regulatory Commission Resident Inspectors' Office Indian Point 3 Nuclear Power Plant

John J. McOscar, Director Division of Resource Management and Administration Region I U.S. Nuclear Regulatory Commission 475 Allendale Road King of Prussia, Pennsylvania 19406-1415

INPO Records Center 700 Galleria Parkway Atlanta, Georgia 30339-5957

OPERATING DATA REPORT

DOCKET NO. DATE COMPLETED BY TELEPHONE IPN-95-086 ATTACHMENT I PAGE 1 of 5 50-286 8-1-95 T. Orlando (914) 736-8340

OPERATING STATUS

Reflects a correction from June 1995

** Weighted Average

1.	Unit Name:Indian Point No. 3 Nuclear	Power Plant					
2.	Reporting Period: July 1						
3.	Licensed Thermal Power (MWt):						
4.	Nameplate Rating (Gross MWe):						
5.	Design Electrical Rating (Net MWe):						
6.	Maximum Dependable Capacity (Gross MWe						
7.	Maximum Dependable Capacity (Net MWe):	965					
8.	If Changes Occur in Capacity Ratings (Items		n 7) Since Last Repo	rt			
	Give Reasons:		· · · · · · · · · · · · · · · · · · ·				
9.							
9. 10.	Power Level to Which Restricted, If Any (Net M Reasons for Restrictions, If Any:	vve):	And the second second				
10.	nousons for nestrictions, if Any.						
		This Month	Yr. to Date	Cumulative			
11.	Hours In Reporting Period	744	5087	165,960			
12.	Number of Hours Reactor Was Critical	726.9		92,687.9			
13.	Reactor Reserve Shutdown Hours	0	0	0			
14.	Hours Generator On-Line	621.4	622.2	90,085.2			
15.	Unit Reserve Shutdown Hours	0	0	0			
16.	Gross Thermal Energy Generated (MWH)	1,419,280	1,424,998 *	255,494,700 *			
17.	Gross Electrical Energy Generated (MWH)	455,500	455,550	79,844,155			
18.	Net Electrical Generated (MWH)	434.533	434,550	76,791,686			
19.	Unit Service Factor	83.5	12.2	54 .3			
20.	Unit Availability Factor	83.5	12.2	54.3			
21.	Unit Capacity Factor (Using MDC Net)	60.5	8.9	49.1**			
22.	Unit Capacity Factor (Using DER Net)	60.5	8,9	48.0			
23.	Unit Forced Outage Rate	14.4	87.7	29.0			
24.	Shutdowns Scheduled Over Next 6 Months (Tune Date and D	tion of Fooble				
Z 4 .	Shutdowns Scheduled Over Next o Months (Type, Date and D	uration of Each):				
25.	If Shut Down At End Of Report Period. Estim	ated Date of Star	tup:				
26.	Units In Test Status (Prior to Commercial Operation):						
	·		cast Achiev	ed			
	INITIAL CRITICALITY						
	INITIAL ELECTRICITY			_			
	COMMERCIAL OPERATION						

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MONTH _	JULY 1995				
DAY AVE	RAGE DAILY POWER LEVEL (MWe-Net)	DAY AVERAGE DAILY POWER LEVEL (MWe-Net)			
1	0	17	868		
2	67	18	878		
3	71	19	897		
4	185	20	68		
5	258	21	109		
6	313	22	567		
7	664	23	966		
8	863	24	966		
9 -	869	25	967		
10	516	26	886		
11	429	27	972		
12	430	28	971		
13	4	29	974		
14	6	30	974		
15	529	31	972		

INSTRUCTIONS:

866

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO.

50-286

UNIT NAME

INDIAN POINT NO. 3

DATE

8-1-95

COMPLETED BY **TELEPHONE**

T. Orlando (914) 736-8340

IPN-95-086

ATTACHMENT I

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REPORT MONTH ___JULY 1995

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR 3	LICENSEE EVENT REPORT #	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
2	950629	F	1.18	А	2	95-012-00	СН	PUMP XX B	MANUAL UNIT TRIP DUE TO THE INABILITY OF NO. 32 MAIN BOILER FEED PUMP TO OPERATE. THIS CAUSED LOW LEVELS IN THE PLANT'S STEAM GENERATORS.
3	950702	Ø	18.35	В	NA	NA	HA	TURBIN	MANUALLY SHUTDOWN THE TURBINE IN ORDER TO PERFORM SURVEILLANCE TEST 3PT-V21, TURBINE GENERATOR OVERSPEED TRIP TEST. ALSO DURING THIS TIME PERIOD, BALANCE MOVES WERE PERFORMED ON THE UNITS MAIN TURBINE GENERATOR IN ORDER TO REDUCE VIBRATIONS.

F: Forced S: Scheduled 2

Reason:

A-Equipment

B-Maintenance or Test

C-Refueling

D- Regulatory Restriction

3

Method

1-Manual

2-Manual Scram

3-Automatic Scram

4-Other (Explain)

Exhibit - Instructions for Preparation of Data **Entry Sheets for Licensee Event Report (LER) File** (NUREG - 0161)

Exhibit - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. **UNIT NAME** DATE COMPLETED BY **TELEPHONE** IPN-95-086 ATTACHMENT I

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50-286 INDIAN POINT NO. 8-1-95 T. Orlando

(914) 736-8340

REPORT MONTH ___JULY 1995

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR 3	LICENSEE EVENT REPORT #	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
4	950710	F	NA	A	NA	NA	SH	INSTRU X	TURBINE RUNBACK FROM 89% POWER TO 50% POWER WHILE SECURING FROM SURVEILLANCE TEST 3PT-V11, OVERTEMPERATURE DELTA T/OVER POWER DELTA T. TEMPERATURE MODULE TM-412D WAS FOUND TO HAVE A FAULTY SIGNAL GENERATOR.
5	950713	F	44.8	A	1	NA	xx	VALVE XX	MANUAL SHUTDOWN DUE TO PRESSURIZER SAFETY RELIEF VALVE PCV-464 INDICATING SIGNS OF LEAKING BY ITS SEAT.
6	950720	F	34.32	В	NA	NA	ED	TRANSF	MANUALLY SHUTDOWN THE TURBINE DUE TO ADVERSE CONDITIONS THAT EXISTED IN THE UNIT AUXILIARY TRANSFORMER. ALSO, DURING THIS PERIOD, ADDITIONAL BALANCE MOVES WERE PERFORMED ON THIS UNIT'S MAIN TURBINE GENERATOR IN ORDER TO REDUCE VIBRATIONS.

F: Forced

S: Scheduled

Reason: A-Equipment **B-Maintenance or Test** C-Refueling **D- Regulatory Restriction**

2

3 Method 1-Manual

2-Manual Scram 3-Automatic Scram 4-Other (Explain)

Exhibit - Instructions for Preparation of Data **Entry Sheets for Licensee** Event Report (LER) File (NUREG - 0161)

Exhibit - Same Source

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SUMMARY OF OPERATING EXPERIENCE

JULY 1995

Indian Point Unit No. 3 was synchronized to the bus for a total of 621.4 hours producing a gross generation of 455,500 Mwe.

On June 29, at 1950 hours, control room operators manually tripped the reactor in response to low levels in the plant's steam generators. The low levels were caused by the inability of No. 32 Main Boiler Feed Pump (MBFP) to adequately provide feedwater flow to the steam generators.

The reactor was brought critical on June 30, at 1853 hours and the unit synchronized to the bus on July 2, at 0111 hours.

On July 2, at 1755 hours, the turbine was manually shutdown for a scheduled outage to perform surveillance test 3PT-V21, <u>Turbine Generator Overspeed Trip Test</u>. Also, during this time period, balance moves were performed on the unit's main turbine generator in order to reduce vibrations. The unit was synchronized to the bus on July 3, at 1216 hours.

On July 10, at 0428 hours, with the unit at approximately 89% reactor power, a turbine runback to approximately 50% reactor power was experienced. During the performance of surveillance test 3PT-V11, Overtemperature Delta T/Overpower Delta T, temperature module TM-412D was found to have a faulty signal generator. While securing from the test, due to the faulty module, an overpower delta T runback occurred. The unit was then stabilized at 50% reactor power.

Also during this event, both pressurizer power operated relief valves (PORVs) cycled due to an elevated reactor coolant temperature and pressure. Evidence indicates they were in the open position for a few seconds. (This is being reported as required by Technical Specification 6.9.1.4.)

Following this event, pressurizer safety valve RC-PCV-464 indicated signs of leaking by its seat. In response to this, the turbine was manually shutdown on July 13, at 0118 hours, and the reactor manually shutdown at 0145 hours. Reactor coolant pressure was then reduced to reseat the valve. After this procedure proved successful, the reactor was brought critical on July 14, at 1524 hours, and the unit synchronized to the bus at 2206 hours.

On July 19, at 1830 hours, the unit achieved full power. At 2000 hours, a controlled load reduction commenced to remove the unit from service. The unit was manually shutdown on July 20, at 0602 hours. This was necessary to perform repairs on the unit auxiliary transformer which had shown evidence of high combustibles and high oxygen levels during routine oil sampling. An investigation showed that a vendor had not properly tightened bolts on the tap changer. Also, during this time period, additional balance moves were performed on the unit's main turbine generator in order to reduce vibrations.

After successful completion of the unit auxiliary transformer repairs and balancing of the main turbine generator, the unit was synchronized to the bus on July 21, at 1621 hours. The unit achieved full power on July 22, at 2359 hours, and remained on line at full power for the remainder of the reporting period.