

**Indian Point 3
Nuclear Power Plant**
P.O. Box 215
Buchanan, New York 10511
914-736-8000



July 10, 1995
IPN-95-076

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
License No. DPR-64
Monthly Operating Report for June 1995

Dear Sir:

The attached monthly operating report, for the month of June 1995, is hereby submitted in accordance with Indian Point 3 Nuclear Power Plant Technical Specification 6.9.1.4.

The Authority is making no commitments in this letter.

Very truly yours,

A handwritten signature in black ink, appearing to read 'L.M. Hill'.

L.M. Hill
Site Executive Officer
Indian Point 3 Nuclear Power Plant

LMH/azk

Attachment

cc: See next page

9507190325 950630
PDR. ADOCK 05000286
R PDR

JE24/1

cc: Thomas T. Martin
Regional Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406-1415

U.S. Nuclear Regulatory Commission
Resident Inspectors' Office
Indian Point 3 Nuclear Power Plant

John J. McOscar, Director
Division of Resource Management and Administration
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406-1415

INPO Records Center
700 Galleria Parkway
Atlanta, Georgia 30339-5957

OPERATING DATA REPORT

DOCKET NO. 50-286
 DATE 7-1-95
 COMPLETED BY T. Orlando
 TELEPHONE (914) 736-8340
 IPN-95-076
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OPERATING STATUS

1. Unit Name: Indian Point No. 3 Nuclear Power Plant
2. Reporting Period: June 1995
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report Give Reasons: _____
9. Power Level to Which Restricted, If Any (Net MWe): _____
10. Reasons for Restrictions, If Any: _____

	This Month	Yr. to Date	Cumulative
11. Hours In Reporting Period	<u>720</u>	<u>4343</u>	<u>165,216</u>
12. Number of Hours Reactor Was Critical	<u>70.9</u>	<u>70.9</u>	<u>91,961.0</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>0.8</u>	<u>0.8</u>	<u>89,463.8</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>169</u>	<u>169</u>	<u>254,069,871</u>
17. Gross Electrical Energy Generated (MWH)	<u>50</u>	<u>50</u>	<u>79,388,655</u>
18. Net Electrical Generated (MWH)	<u>17</u>	<u>17</u>	<u>76,357,153</u>
19. Unit Service Factor	<u>0.1</u>	<u>0</u>	<u>54.1</u>
20. Unit Availability Factor	<u>0.1</u>	<u>0</u>	<u>54.1</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0</u>	<u>0</u>	<u>49.0*</u>
22. Unit Capacity Factor (Using DER Net)	<u>0</u>	<u>0</u>	<u>47.9</u>
23. Unit Forced Outage Rate	<u>99.9</u>	<u>99.9</u>	<u>29.0</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each): _____

25. If Shut Down At End Of Report Period. Estimated Date of Startup: 7/2/95

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

* Weighted Average

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-286
 UNIT IP-3
 DATE 7-1-95
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MONTH JUNE 1995

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>1</u>
30	<u>0</u>
31	<u>-----</u>

INSTRUCTIONS:

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-286
 UNIT NAME INDIAN POINT NO. 3
 DATE 7-1-95
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 TELEPHONE (914) 736-8340
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REPORT MONTH JUNE 1995

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR 3	LICENSEE EVENT REPORT #	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
1	930226	F	691.02	B	1	93-005-02	IE	INSTRU X	THE UNIT WAS REMOVED FROM SERVICE IN ORDER TO PERFORM TESTING ON THE PLANT'S AMSAC SYSTEM.
2	950629	F	28.17	A	2	95-0XX (REPORT SUBMITTAL DATE 7/29/95)	CH	PUMP XX B	MANUAL UNIT TRIP DUE TO THE INABILITY OF NO. 32 MAIN BOILER FEED PUMP TO OPERATE. THIS CAUSED LOW LEVELS IN THE PLANT'S STEAM GENERATORS.

1
 F: Forced
 S: Scheduled

2
 Reason:
 A-Equipment
 B-Maintenance or Test
 C-Refueling
 D- Regulatory Restriction

3
 Method
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

4
 Exhibit - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File
 (NUREG - 0161)

5
 Exhibit - Same Source

SUMMARY OF OPERATING EXPERIENCE

JUNE 1995

Indian Point Unit No. 3 was synchronized to the bus for a total of 0.82 hours producing a gross generation of 50 Mwe.

During the course of the month of May, plant operators received and acknowledged alarms indicating minor reactor vessel inner and outer seal leakage. After this leak was confirmed, operators began to bring the plant unit to the cold shutdown mode in order to facilitate reactor vessel seal replacement. Cold shutdown was reached on May 29, 1995 at 1530 hours.

When reactor vessel seal replacement repairs and testing were successfully completed, the unit entered the hot shutdown mode on June 17, at 0345 hours. Normal reactor coolant system operating temperature and pressure was achieved on June 19, at 0635 hours, in preparation for plant restart.

The reactor was brought critical on June 27, at 0205 hours, and the unit synchronized to the bus on June 29, at 1901 hours.

On June 29, at 1950 hours, control room operators manually tripped the reactor in response to low levels in the plant's steam generators. The low levels were caused by the inability of No. 32 Main Boiler Feed Pump (MBFP) to adequately provide feedwater flow to the steam generators.

The reactor was brought critical on June 30, at 1853 hours.

The unit remained off-line for the remainder of the reporting period.