



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 156  
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Power Authority of the State of New York (the licensee) dated September 16, 1994, as supplemented November 29, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 156, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Michael J. Case, Acting Director  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 22, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 156

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

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## 6.0 ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

6.1.1 The Resident Manager shall be responsible for overall facility operation. During periods when the Resident Manager is unavailable, one of the three General Managers will assume his responsibilities. In the event all four are unavailable, the Resident Manager may delegate this responsibility to other qualified supervisory personnel.

### 6.2 ORGANIZATION

#### 6.2.1 Facility Management and Technical Support

Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.

- a) Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the Updated FSAR.
- b) The Resident Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
- c) The Executive Vice President and Chief Nuclear Officer shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.

- c. Review of all proposed changes to the Operating License and Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL.
- f. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Resident Manager, who will forward the report to the Executive Vice President and Chief Nuclear Officer, Vice President Regulatory Affairs and Special Projects, and the Chairman of the Safety Review Committee.
- g. Review of all reportable events.
- h. Review of facility operations to detect potential nuclear safety hazards.
- i. Performance of special reviews, investigations or analyses and reports thereon as requested by the Resident Manager or the Chairman of the Safety Review Committee (SRC).
- j. Review of the Plant Security Plan and implementing procedures annually.
- k. Review of the Emergency Plan and implementing procedures annually.
- l. Review of every unplanned onsite release of radioactive material to the environs including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Resident Manager and to the Safety Review Committee.

#### AUTHORITY

6.5.1.7 The Plant Operating Review Committee shall:

- a) Recommend to the Resident Manager approval or disapproval of items considered under 6.5.1.6(a) through (e) above.
- b) Render determinations with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question, as defined in 10 CFR 50.59.

- c) Provide notification within 24 hours to the Chairman of the SRC and the Executive Vice President and Chief Nuclear Officer of disagreement between the PORC and the Resident Manager; however, the Resident Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

#### RECORDS

6.5.1.8 The Plant Operating Review Committee shall maintain minutes of each meeting and copies shall be provided to the Chairman of the SRC and Executive Vice President and Chief Nuclear Officer.

#### 6.5.2 SAFETY REVIEW COMMITTEE (SRC)

#### FUNCTION

6.5.2.1 The SRC shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical engineering
- h. Electrical engineering
- i. Administrative controls and quality assurance practices
- j. Environment
- k. Civil/Structural Engineering
- l. Emergency Planning
- m. Nuclear Licensing
- n. Other appropriate fields associated with the unique characteristics of a nuclear power plant.

#### CHARTER

6.5.2.2 The conduct of the SRC will be in accordance with a charter approved by the Executive Vice President and Chief Nuclear Officer. The charter will define the SRC's authority and establish the mechanism for carrying out its responsibilities.

#### MEMBERSHIP

6.5.2.3 The SRC shall be composed of at least six individuals including a Chairman and a Vice Chairman. Members shall be appointed by the Vice President Regulatory Affairs and Special Projects and approved by the Executive Vice President and Chief Nuclear Officer. SRC members and alternates shall have an academic degree in engineering or a physical science, or the equivalent, and shall have a minimum of five years technical experience in one or more areas listed in 6.5.2.1.

#### ALTERNATES

6.5.2.4 Alternates for the Chairman, Vice Chairman and members may be appointed in writing by the Vice President Regulatory Affairs and Special Projects and approved by the Executive Vice President and Chief Nuclear Officer.

#### CONSULTANTS

6.5.2.5 Consultants may be used as determined by the SRC Chairman and as provided for in the charter.

#### MEETING FREQUENCY

6.5.2.6 The SRC shall meet at least once per six months.

#### QUORUM

6.5.2.7 A quorum shall consist of at least a majority of the appointed individuals (or their alternates) and the Chairman (or the designated alternate). No more than two alternates may participate as SRC voting members at any one time. No more than a minority of the quorum shall have direct line responsibility for the operation of the plant.



## REVIEW

### 6.5.2.8 The SRC shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications of this Operating License.
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. All REPORTABLE EVENTS.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the Plant Operating Review Committee.

## AUDITS

### 6.5.2.9

Audits of facility activities shall be performed under the cognizance of the SRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or methods of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B," 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 12 months.
- f. The Facility Security Plan including the Safeguards Contingency Plan and implementing procedures at least once per 12 months.
- g. Any other area of facility operation considered appropriate by the SRC or the Executive Vice President and Chief Nuclear Officer.
- h. The Facility Fire Protection Program and implementing procedures at least once per two years.
- i. A fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.
- k. The radiological environmental monitoring program and the results thereof at least once per 12 months.

1. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.
- m. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.

#### AUTHORITY

- 6.5.2.10 The SRC shall advise the Executive Vice President and Chief Nuclear Officer on those areas of responsibility specified in Sections 6.5.2.8 and 6.5.2.9.

#### RECORDS

- 6.5.2.11 Records will be maintained in accordance with ANSI 18.7-1972. The following shall be prepared and distributed as indicated below:

- a. Minutes of each SRC meeting shall be prepared and forwarded to the Executive Vice President and Chief Nuclear Officer within 30 days after the date of the meeting.
- b. Reports of reviews encompassed by Section 6.5.2.8 above shall be prepared and forwarded to the Executive Vice President and Chief Nuclear Officer within 30 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.9 above, shall be forwarded to the Executive Vice President and Chief Nuclear Officer and to the management positions responsible for the areas audited within 30 days after the completion of the audit.

#### 6.6 REPORTABLE EVENT ACTION

- 6.6.1 The following actions shall be taken for REPORTABLE EVENTS:
- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and

- b. Each REPORTABLE EVENT shall be reviewed by the PORC and a report submitted by the Resident Manager to the Executive Vice President and Chief Nuclear Officer, Vice President Regulatory Affairs and Special Projects, and the Chairman of the SRC.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The reactor shall be shut down and reactor operation shall only be resumed in accordance with the provisions of 10 CFR 50.36(c)(1)(1).
- b. The Safety Limit Violation shall be reported immediately to the Commission. The Executive Vice President and Chief Nuclear Officer, Vice President Regulatory Affairs and Special Projects, and the Chairman of the SRC will be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared by the PORC. This report shall describe (1) applicable circumstances preceding the occurrences, (2) effects of the occurrence upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the Executive Vice President and Chief Nuclear Officer, the Vice President Regulatory Affairs and Special Projects, and the Chairman of the SRC by the Resident Manager.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.
- b. Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Process Control Program implementation.
- g. Offsite Dose Calculation Manual implementation.