



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 76 TO FACILITY OPERATING LICENSE NO. DPR-64

POWER AUTHORITY OF THE STATE OF NEW YORK

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

DOCKET NO. 50-286

INTRODUCTION

By letter dated February 9, 1987, The Power Authority of the State of New York (the licensee) seeks to amend Section 4.9 of the Technical Specifications appended to the operating license. The proposed change modifies the definition of Tube Inspection to reflect the definition recommended in Generic Letter 85-02. In addition, it is proposed to delete historical references to the tube plugging limit of 63% degradation which were valid for Cycle 4 only. Table 4.9-1 is revised to reflect that a defective tube may be repaired by plugging or sleeving. Additionally, the Bases for Section 4.9 has been revised and updated to provide new information for wastage-type defects.

The licensee also requested a proposed change that would permit the resumption of power operation without prior NRC approval subsequent to a steam generator inspection whose results have been classified as C-3 (i.e., all tubes were inspected per the provisions of Table 4.9-1 of the Technical Specifications).

DISCUSSION AND EVALUATION

The Indian Point 3 Technical Specifications currently require sample inspections from the hot leg side of the tubesheet to the top support on the cold leg side. In light of the degradation on the cold leg side, full length tube inspections will be performed for the initial 3% sample defined in Table 4.9-1. Additional sample inspections may be limited to a partial length inspection concentrating on those locations where degradation has been found. This definition of tube inspection is that recommended by Generic Letter 85-02, "Staff Recommended Actions Stemming from NRC Integrated Program for the Resolution of Unresolved Safety Issues Regarding Steam Generator Tube Integrity".

For Cycle 4 only, the plugging limit was increased to 63% degradation from 40% degradation. Upon completion of Cycle 4 operations the tube plugging limit reverted to 40%. The references to the 63% tube plugging limit in Section 4.9 are being deleted for the sake of clarity since they no longer apply.

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Table 4.9-1, which contains required actions depending upon tube inspection results, has been revised to reflect that a defective tube may be repaired or plugged. Repair by sleeving of defective tubes was previously permitted by License Amendment No. 47. This change constitutes a clarification to reflect a previously approved alternative to plugging defective tubes.

Bases Section 4.9 for steam generators has been revised and updated to provide a current rationale for Technical Specification 4.9. Historical information was either deleted or updated. Statements in the Bases section addressing wastage-type defects have been revised to reflect information acquired during the steam generator inspection performed during the Cycle 4/5 refueling outage and subsequent analyses. These changes are an improvement to the Bases supporting the steam generator surveillance requirements.

The staff concludes that the incorporation of these changes will not decrease the integrity of the steam generator tubes. The voluntary adoption of tube inspection by a full "U" rather than a "J" as currently defined in the Westinghouse Standard Technical Specification will result in improved inspections and will ensure the continued inspection of the cold leg side, where degradation has been previously identified. Further, the staff concurs in the elimination of the historical reference to the 63% plugging criterion which was valid for Cycle 4 only. The proposed revisions dealing with wastage and sleeving are basically clarifications and are therefore acceptable.

In addition to the above acceptable changes, the licensee proposed to delete the provision for approval prior to resuming power operation subsequent to a steam generator inspection whose results have been classified as C-3. The deletion of this requirement is not acceptable. The Indian Point 3 Steam Generator is one in which severe tube degradation has occurred. Because of this, close staff review is required. There is no basis for removal of the approval requirement at this time.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 7, 1987

PRINCIPAL CONTRIBUTOR:

D. Sellers