



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

POWER AUTHORITY OF THE STATE OF NEW YORK

DOCKET NO. 50-286

INDIAN POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 39
License No. DPR-64

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Power Authority of the State of New York (the licensee) dated June 15, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulation;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

8110230622 811008
PDR ADOCK 05000286
PDR

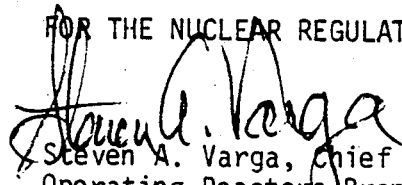
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-64 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 39, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective 21 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 8, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 39

FACILITY OPERATING LICENSE NO. DPR-64

DOCKET NO. 50-286

Revise Appendix A as follows:

Remove Pages

6-1
6-2
6-3
6-8
6-9
6-11
6-12

Insert Pages

6-1
6-2
6-3
6-8
6-9
6-11
6-12

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Resident Manager shall be responsible for overall facility operation. During periods when the Resident Manager is unavailable, the Superintendent of Power will assume his responsibilities. In the event both are unavailable, the Resident Manager may delegate this responsibility to other qualified supervisory personnel. The Resident Manager reports directly to the Sr. Vice President Nuclear Generation, as shown in Fig. 6.2-1.

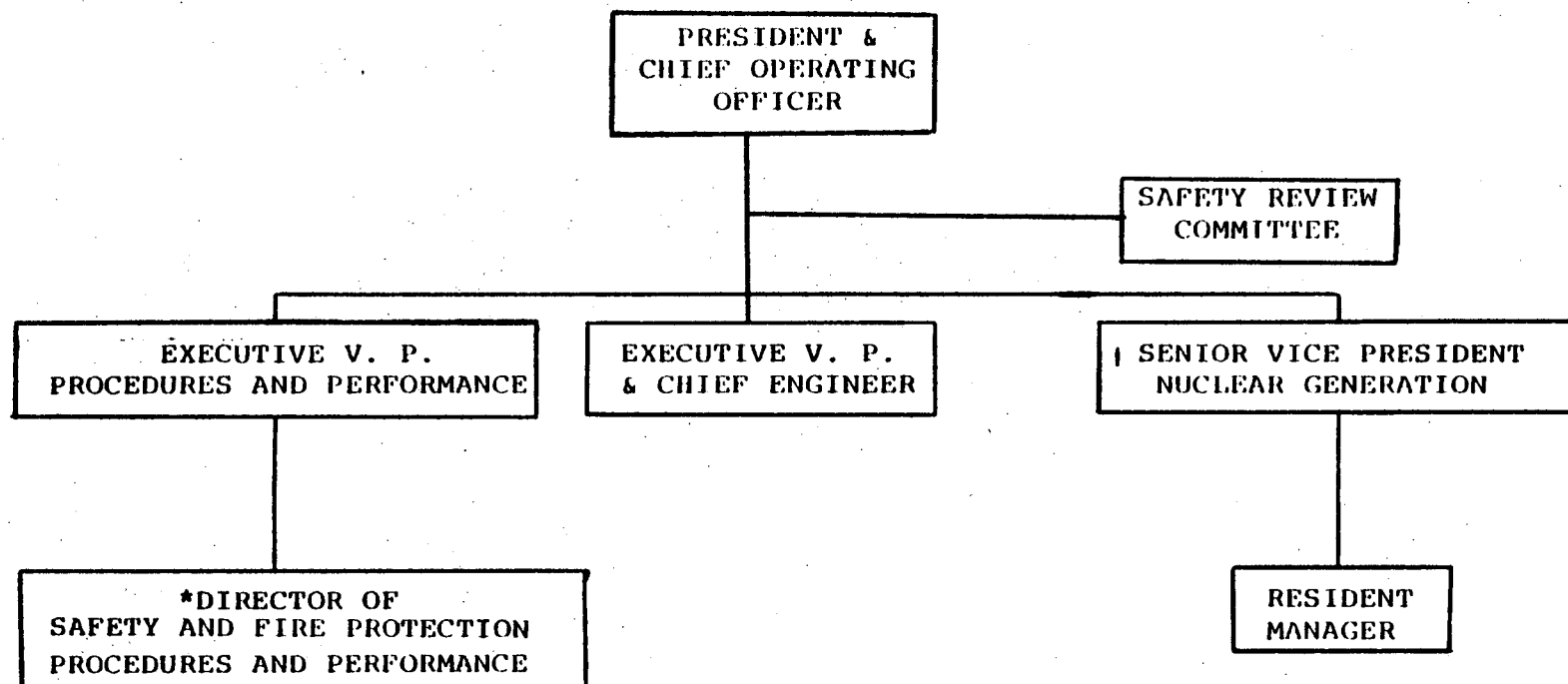
6.2 ORGANIZATION OFFSITE

6.2.1 The offsite organization for plant management and technical support shall be as shown on Figure 6.2-1.

PLANT STAFF

6.2.2 The plant organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one Licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. ALL CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A Fire Brigade of at least five members shall be maintained on site at all times. This excludes four members of the minimum shift crew necessary for safe shutdown of the plant and any personnel required for other essential functions during a fire emergency. During periods of cold shutdown the Fire Brigade will exclude two members of the minimum shift crew.

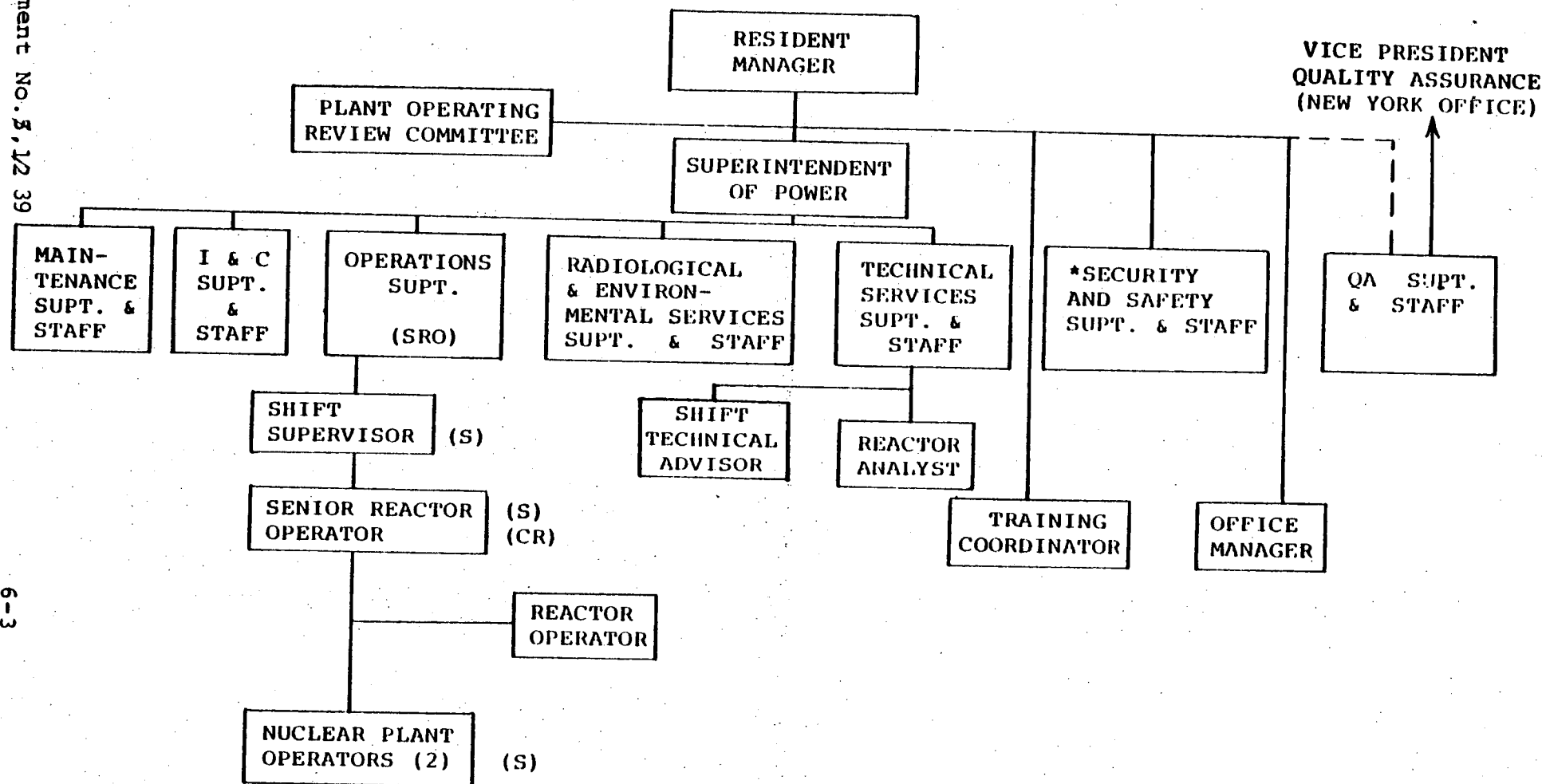


†The Senior Vice President Nuclear Generation is responsible for administering the fire protection program.

*The Director of Safety and Fire Protection is the off-site management position responsible for assessing the effectiveness of the fire protection programs by conducting reviews and audits.

FIGURE 6.2-1

MANAGEMENT ORGANIZATION CHART
INDIAN POINT 3 NUCLEAR POWER PLANT



(S) Continuous Coverage
 (CR) Control Room
 (SRO) Senior Reactor Operator

FIGURE 6.2-2

POWER AUTHORITY OF THE STATE OF NEW YORK
 INDIAN POINT 3 NUCLEAR POWER PLANT
 PLANT STAFF ORGANIZATION

*Responsibility for performance and monitoring of the fire protection program.

RECORDS

6.5.1.8 The Plant Operating Review Committee shall maintain minutes of each meeting and copies shall be provided to the Chairman of the SRC and Senior Vice President-Nuclear Generation.

6.5.2 SAFETY REVIEW COMMITTEE (SRC)

FUNCTION

6.5.2.1 The SRC shall collectively have the competence required to review problems in the following areas:

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical engineering
- h. Electrical engineering
- i. Administrative controls and quality assurance practices
- j. Environment
- k. Civil/Structural Engineering
- l. Emergency Planning
- m. Nuclear Licensing
- n. Other appropriate fields associated with the unique characteristics of a nuclear power plant.

MEMBERSHIP

6.5.2.2 The SRC shall be composed of the following voting members:

Chairman:	Director-Nuclear Design and Analysis
Vice-Chairman:	Vice President-Quality Assurance
Member:	Manager-Nuclear Technical Support
Member:	Manager-Radiological Health and Safety
Member:	Senior Vice President-Nuclear Generation
Member:	Director-Electrical Design and Analysis
Member:	Director of Environmental Programs
Member:	Director-Civil/Structural Design and Analysis
Member:	Senior Inservice Inspection Engineer-Nuclear Technical Support
Member:	Manager-Nuclear Licensing
Member:	Senior Nuclear Engineer-Design and Analysis
Member:	Director-Mechanical Design and Analysis
Member:	Manager-Nuclear Operations

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of facility operation following initial fuel loading and at least once per six months, thereafter.

QUORUM

6.5.2.6 A quorum of SRC shall consist of the Chairman or Vice-Chairman and six members, including alternates. No more than a minority of the quorum shall have a direct line responsibility for the operation of the plant.

AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the SRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 24 months.
- f. The Facility Security Plan and implementing procedures at least once per 24 months.
- g. Any other area of facility operation considered appropriate by the SRC or the President.
- h. The Facility Fire Protection Program and implementing procedures at least once per two years.
- i. A fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.

AUTHORITY

6.5.2.9 The SRC shall report to and advise the President and Chief Operating Officer (to be referred to in the Tech Spec. as the President) on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records will be maintained in accordance with ANSI 18.7-1972. The following shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the President within 14 days after the date of the meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the President within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the President and to the management positions responsible for the areas audited within 30 days after completion of the audit.

CHARTER

6.5.2.11 Conduct of the committee will be in accordance with a charter, approved by the President, setting forth the mechanism for implementation of the committee's responsibilities and authority.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hours notification to the Commission shall be reviewed by the PORC and a report submitted by the Resident Manager to the Chairman of the SRC and Senior Vice President-Nuclear Generation.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The reactor shall be shut down and reactor operation shall only be resumed in accordance with the provisions of 10 CFR 50.36(c)(1)(i).
- b. The Safety Limit violation shall be reported immediately to the Commission. The Chairman of the SRC and Senior Vice President-Nuclear Generation will be notified within 24 hours.