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James Knubel
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November 13, 1997
IPN-97-155

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

SUBJECT: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
License No. DPR-64
**Supplement to the Proposed Technical Specification Change Regarding
Quarterly Inservice Testing of ASME Code Class 1, 2 and 3 Pumps and
Valves**

Reference : NYPA letter to the NRC dated January 2, 1997 (IPN-97-001), "Proposed
Changes to Technical Specifications Regarding Quarterly Inservice Testing of
ASME Code Class 1, 2 and 3 Pumps and Valves."

Dear Sir:

In the referenced letter, the Authority submitted the proposed change to the Indian Point 3 Technical Specifications (TS) regarding the quarterly inservice testing of the ASME Code Class 1, 2 and 3 pumps and valves. The justification for the change was based on the NRC guidance and recommendations provided in NUREG-1366, "Improvements to Technical Specifications Surveillance Requirements."

NUREG-1366 included the recommendation to change the surveillance test frequency for auxiliary feedwater (AFW) pumps to quarterly on a staggered test basis. Conducting the tests on a staggered basis will permit system testing monthly, while each AFW pump is tested quarterly. The intent of this change is to be consistent with the Standard Technical Specifications (STS) and the requirements of the ASME Code, however, because of the definition of "staggered test basis," the STS give the frequency as "monthly" on a staggered basis.

The TS change for the AFW pumps was inadvertently submitted with the referenced letter indicating a quarterly testing frequency. The TS change indicating a monthly test frequency on a staggered basis (i.e. one pump per month, so that each pump is tested once during a 3 month period) is included in Attachment 1. This TS change is consistent with the evaluation and "no significant hazards consideration" enclosed with the referenced letter. This change is in accordance with the ASME Code, and, as stated on page 2 of 3 of attachment 2

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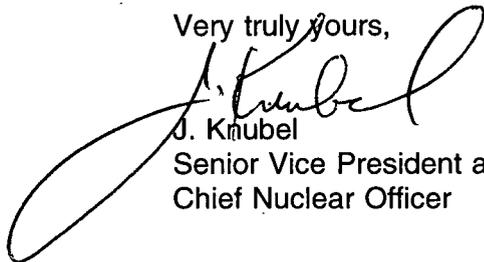
to the referenced letter, is "compatible" with plant operating experience, and consistent with the guidance provided in NUREG-1366 and approved by the NRC in the STS.

In accordance with 10 CFR 50.91, a copy of this letter and attachment are being submitted to the designated New York State official.

The Authority is not making any new commitments in this submittal.

If you have any questions regarding this submittal, please contact Mr. Ken Peters at (914) 736-8029.

Very truly yours,



J. Krubel
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Attachments: as stated

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