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New York Power Authority

James Knubel
Senior Vice President and
Chief Nuclear Officer

June 25, 1997
IPN-97-082

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

SUBJECT: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
License No. DPR-64
Proposed Technical Specification Change Regarding
Control Rod Misalignment and Rod Position Indication

References:
1. NYPA letter to the NRC (IPN-97-010) dated January 27, 1997, "Proposed Technical Specification Change Regarding Control Rod Misalignment and Rod Position Indication."
2. NYPA letter to the NRC (IPN-97-064) dated May 16, 1997, "Supplement to the Proposed Technical Specification Change Regarding Control Rod Misalignment and Rod Position Indication."

Dear Sir:

This application for amendment to the Indian Point 3 Technical Specifications (TS) proposes to amend Sections 3.10.5 and the associated Bases. The proposed amendment would permit changing the indicated control rod misalignment from the current limit of ±12 steps to an indicated misalignment of ±18 steps when the core power is less than or equal to 85% of rated thermal power (RTP). For core power above 85% of RTP, the proposed amendment would permit ±12 steps misalignment for less than or equal to 212 steps withdrawn and +17, -12 steps for greater than 212 steps withdrawn taking into account the physical geometry of the core (dead region). The proposed change is bounded by an evaluation performed by Westinghouse (WCAP-14668) which was submitted in Reference 1. In Reference 1, the Authority submitted a proposed TS amendment to increase the control rod misalignment to ±24 steps for core power less than or equal to 85% RTP and ±12 steps above 85% RTP with certain other considerations. In Reference 2, the Authority submitted a supplement to Reference 1 that would allow control rod misalignment of ±18 steps when the core power is less than or equal to 85% of RTP and ±12 steps above 85% of RTP. It is anticipated that the proposed TS amendment as submitted in Reference 2 will be granted by the NRC prior to the approval of this submittal. Therefore, the current submittal will then only address the control rod misalignment of +17, -12 steps above 85% of RTP for greater than 212 steps. The conclusions of the safety

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evaluation is not changed as the current proposed change is bounded by the Westinghouse analysis submitted in Reference 1.

The proposed TS change pages are included in Attachment I. The safety evaluation is included in Attachment II.

Enclosed for filing is the signed original of a document entitled, "Application for Amendment to Operating License," together with Attachments I and II, comprising a statement of the proposed change to the Technical Specifications and the safety evaluation.

The commitments associated with this submittal are in Attachment III.

In accordance with 10 CFR 50.91, a copy of this application and the associated attachments are being submitted to the designated New York State official.

If you have any questions regarding this letter, please contact Ms. C. D. Faison at (914) 736-8029.

Very truly yours,

J. Knubel
Senior Vice President and
Chief Nuclear Officer

Attachments: as stated

cc: U.S. Nuclear Regulatory Commission
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Resident Inspector's Office
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