

123 Main Street
White Plains, New York 10601
914 681.6840
914 287.3309 (FAX)



James Knubel
Senior Vice President and
Chief Nuclear Officer

May 16, 1997
IPN-97-064

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

SUBJECT: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
License No. DPR-64
**Supplement to the Proposed Technical Specification Change Regarding
Control Rod Misalignment and Rod Position Indication**

Reference 1: NYPA letter to the NRC (IPN-97-010) dated January 27, 1997, " Proposed
Technical Specification Change Regarding Control Rod Misalignment and Rod
Position Indication."

Dear Sir:

In the referenced submittal, the Authority proposed an amendment to the
Indian Point 3 Technical Specifications to change the indicated control rod misalignment from
 ± 12 steps to an indicated misalignment of ± 24 steps when the core power is less than or
equal to 85% of rated thermal power (RTP) and ± 12 steps above 85% of RTP with certain
other considerations. The proposed change was based on an evaluation performed by
Westinghouse (WCAP-14668).

Indian Point 3 is in a refueling outage that started on May 14, 1997. The proposed TS
change submitted in Reference 1 for control rod misalignment of ± 24 steps when the core
power is less than or equal to 85% of RTP and ± 12 steps above 85% of RTP with certain
other considerations needed longer review time by the NRC. The approval of this proposed
amendment may not be granted in time to be implemented to support the current startup
schedule. In order to expedite the NRC review and approval, the Authority is submitting this
revised TS change. The revised TS change is bounded by the Westinghouse analysis
submitted in Reference 1. It will permit control rod misalignment of ± 18 steps when the core
power is less than or equal to 85% of RTP and ± 12 steps above 85% of RTP. The
conclusions of the safety evaluation, Westinghouse Analysis (WCAP-14668) and the no
significant hazards consideration evaluation is not changed as the revised change is bounded
by the Westinghouse analysis submitted in Reference 1.

The revised and the marked up TS pages are included in Attachment I. The revised
safety evaluation is included in Attachment II.

The Authority respectfully requests that the proposed amendment be granted by

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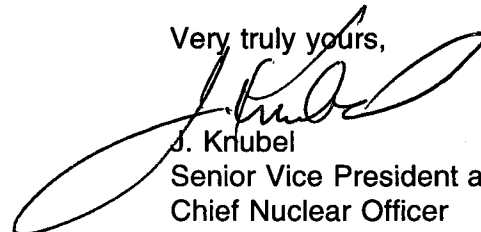
July 10, 1997 to support the current startup schedule.

The Authority is not making any new commitments in this submittal.

In accordance with 10 CFR 50.91, a copy of this letter and the associated attachments are being submitted to the designated New York State official.

If you have any questions, please contact Ms. C. D. Faison.

Very truly yours,



J. Knubel
Senior Vice President and
Chief Nuclear Officer

Attachments: as stated

cc: Mr. Hubert J. Miller
Regional Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Resident Inspector's Office
Indian Point Unit 3
U.S. Nuclear Regulatory Commission
P.O. Box 337
Buchanan, NY 10511

Mr. F. William Valentino, President
New York State Energy, Research, and
Development Authority
Corporate Plaza West
286 Washington Avenue Extension
Albany, NY 12203-6399

Mr. George F. Wunder, Project Manager
Project Directorate I-1
Division of Reactor Projects I/II
U.S. Nuclear Regulatory Commission
Mail Stop 14 B2
Washington, DC 20555