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William J. Cahill, Jr.
Chief Nuclear Officer

January 13, 1997
IPN-97-006

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Subject: Indian Point 3 Nuclear Power Plant
Docket No. 50-286
Proposed Change to Technical Specifications
Regarding Incorporation of 10 CFR 50, Appendix J, Option B

Dear Sir:

This application for amendment to the Indian Point 3 Technical Specifications proposes to revise Technical Specification 4.4, "Containment Tests," requirements for containment leakage testing to add several containment isolation valves to Table 4.4-1 and to implement 10 CFR 50, Appendix J, Option B which allows use of performance based surveillance frequencies for Type A, B, and C tests, rather than predetermined fixed intervals. Implementation of Option B to Primary Containment Leakage Rate Testing Program, will be controlled using the Primary Containment Leakage Rate Testing Program. A new Administrative Controls section, "6.14 Primary Containment Leakage Rate Testing Program," has been added to the Technical Specifications to describe this program.

This submittal is a cost beneficial licensing action because implementation will save in excess of the \$100,000 guideline with no significant reduction in public health and safety. The Power Authority requests review and approval of this application prior to April 1, 1997 in order to adopt these changes prior to the upcoming refueling outage currently scheduled for April 1997.

Enclosed for filing is the signed original of a document entitled, "Application for Amendment to Operating License." Attachment I to the application contains the proposed changes to the Technical Specifications. Attachment II contains the associated Safety Evaluation and the implementation plan, required by Section V.B.2 of 10 CFR 50, Appendix J, for implementing the new leakage rate testing requirements. Commitments made in this application are attached.

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In accordance with 10 CFR 50.91, a copy of this application and the associated attachments are being submitted to the designated New York State official.

If you have any questions, please contact Mr. K. Peters.

Very truly yours,



William J. Cahill, Jr.
Chief Nuclear Officer

Attachments

cc: U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Resident Inspector's Office
Indian Point Unit 3
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Mr. George F. Wunder, Project Manager
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Attachment III
List of Commitments

Number	Commitment	Due
IPN-97-006-01	<p>The Authority commits to the development of a Containment Leakage Rate Testing Program and implementing procedures that will establish the administrative controls to implement 10 CFR 50, Appendix J, Option B. The administrative controls will conform with Regulatory Guide 1.163, dated September 1995, including the approved documents NEI 94-01, dated July 26, 1995, and, with two exceptions (discussed in the administrative Technical Specification for the Option B Implementation Plan), ANSI/ANS 56.8 -1994. The Containment Leakage Rate Testing Program, procedures for performing testing per the new standard, procedures for tracking test results and acceptance criteria will be completed and issued for implementation prior to extending any test interval (implementation of Option B for Type A testing may proceed separately from Type B and C implementation). Completion of these activities and implementation of procedures may take place prior to receipt of the Technical Specification change since they will not be used until the outage and may always be withdrawn if the change is not approved. Completion of some portions of these activities and implementation of procedures may take place after the implementation of the Technical Specification change since the existing test procedures, acceptance criteria and test frequency remain in place until implementation and these are consistent with the revised Technical Specification.</p>	Prior to use