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Executive Vice President
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December 5, 1994
IPN-94-152

U.S. Nuclear Regulatory Commission
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Subject: **Indian Point 3 Nuclear Power Plant**
Docket No. 50-286
Emergency Core Cooling System Evaluation Changes

Reference: Westinghouse letter, J. R. Gasperini to L. Hill, dated October 27, 1994,
"SBLOCTA Axial Nodalization" (INT-94-219).

Dear Sir:

This letter describes recent changes to the Indian Point 3 emergency core cooling system (ECCS) evaluation model, and how these changes affect the peak cladding temperature (PCT). As defined in 10 CFR 50.46(a)(3)(i), the changes in the small break loss of coolant accident (LOCA) analysis described below are significant (PCT change greater than 50°F). This letter fulfills the 30 day reporting requirement.

The Authority has reviewed the small and large break LOCA evaluation model changes described in the referenced letter. The letter describes a change in PCT of -237°F for a small break LOCA. This PCT decrease is the result of several factors, namely an improved axial nodalization methodology, a revision to the rod internal pressure model, recent code revisions and error corrections. Normally these items would be addressed separately in the 10 CFR 50.46 year end summary report and their individual effects on the PCT would be listed. However, the effect of these items on the PCT has been combined into one estimate, as the revised code used to determine the new PCT implicitly addressed all of these elements.

The Authority has reviewed the previously described changes and determined that the Indian Point 3 PCT is well below the maximum fuel cladding temperature limit of 2200°F and that Indian Point 3 continues to comply with 10 CFR 50.46.

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No commitments are being made by the Authority in this submittal. If you have any questions, please contact Ms. C. D. Faison.

Very truly yours,



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cc: U.S. Nuclear Regulatory Commission
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