123 Main Street White Plains, New York 10601 914-681-6840 914-287-3309 (FAX)



William J. Cahill, Jr. **Executive Vice President** and Chief Nuclear Officer

September 29, 1994 IPN-94-122

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Mail Station P1-137 Washington, DC 20555

Subject:

Indian Point 3 Nuclear Power Plant

Docket No. 50-286

Schedular Exemption from Plant Specific Requirements Associated with 10 CFR 50, Appendix J. Section III.D.3 and Proposed Changes to the Technical Specifications Regarding the Leak Rate Testing Interval for Residual Heat Removal Containment Isolation Valves

Reference:

NRC Letter, R. A. Capra to R. E. Beedle, "Issuance of an Exemption from the

Requirements of 10 CFR Part 50, Appendix J, Paragraph III.D.3." dated

February 19, 1993.

This application requests a one time schedular exemption from the Indian Point Unit 3 (IP3) specific requirements associated with 10 CFR Part 50-Appendix J-Type C testing and an amendment to the IP3 Technical Specifications. The exemption and amendment would permit a one time deferral of the leak rate test for Residual Heat Removal (RHR) containment isolation valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870. The leak rate test for these valves would be deferred until the next refueling outage, currently scheduled for the spring of 1996.

In accordance with Technical Specification 4.4.E.1 and the exemption granted by the referenced letter, containment isolation valves are currently required to be leak rate tested at intervals no greater than 30 months. As a result of the current extended Restart and Continuous Improvement Outage, this 30 month interval will be exceeded for some valves before the next refueling outage, scheduled for the spring of 1996. Except for RHR valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870, all containment isolation valves can be tested prior to exceeding the 30 month requirement. However, the procedures for testing valves AC-732, AC-741, AC-MOV-743, AC-MOV-744, and AC-MOV-1870 require that the RHR system be out of service. It would be impractical to remove the RHR system from service during the current outage because the system is needed to remove reactor decay heat. Therefore, it would be impractical to test the RHR system containment isolation valves prior to exceeding the 30 month requirement. This application will allow a one time extension to the 30 month leak rate test interval for valves AC-732, AC-741, AC-MOV-743, AC-MOV-744 and AC-MOV-1870.

9410040190 940929 PDR ADDCK 05000286

The signed original of the Application for Amendment to Operating License is enclosed for filing. Attachment I to this application contains the request for exemption from IP3 specific requirements associated with 10 CFR Part 50-Appendix J-Type C testing. Attachments II and III of this application contain the proposed change to the Technical Specifications and the associated Safety Evaluation.

In accordance with 10 CFR 50.91, a copy of this application and the associated attachments is being submitted to the designated New York State official.

This letter contains no new commitments. If you have any questions regarding this letter, please contact Mr. P. Kokolakis.

Very truly yours,

William J. Cahill, Sr. _____ Executive Vice President and

Chief Nuclear Officer Nuclear Generation

att:

as stated

cc:

U.S. Nuclear Regulatory Commission

475 Allendale Road

King of Prussia, PA 19406

Resident Inspector's Office

Indian Point Unit 3

U.S. Nuclear Regulatory Commission

P.O. Box 337

Buchanan, NY 10511

Ms. Donna Ross New York State Energy Office Division of Policy Analysis & Planning Empire State Plaza Building Number 2 - 16th Floor Albany, NY 12223

Mr. Nicola F. Conicella, Project Manager Project Directorate I-1 Division of Reactor Projects I/II U.S. Nuclear Regulatory Commission Mail Stop 14 B2 Washington, DC 20555