



February 3, 1994
IPN-94-014

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

Subject: **Indian Point 3 Nuclear Power Plant
Docket No. 50-286
Proposed Changes to Technical Specifications
Regarding Extension of Calibration Intervals for Engineered
Safety Features Actuation Systems (ESFAS)/Indicating Instrument
Circuits to Accommodate a 24 Month Operating Cycle**

Reference: (1) NYPA letter to NRC, dated February 18, 1993 (IPN-93-007), "Proposed Changes to Technical Specifications Regarding Extension of Surveillance Test and Calibration Intervals for Reactor Protection System (RPS) to Accommodate a 24 Month Operating Cycle."

Dear Sir:

Starting with cycle nine (that began in August 1992), Indian Point 3 began operating on 24 month cycles, instead of the previous 18 month cycles. To accommodate a 24 month operating cycle, this application for amendment to the Indian Point 3 Technical Specifications proposes to change the calibration frequency for many of the Engineered Safety Features Actuation Systems (ESFAS) circuits and the turbine trip auto stop oil pressure channels listed in Indian Point 3 Technical Specification Table 4.1-1.

Generally, the proposed changes to Table 4.1-1 are shown as changing the notation "18M" (at least once per 18 months) to "24M" (at least once per 24 months). The instrument channels whose calibration frequency will be changed by this application are:

- reactor coolant temperature (Item 4),
- steam generator level (Items 10 and 28a),
- containment pressure (Item 14),
- steam line pressure (Item 18),
- turbine first stage pressure (Item 19),
- turbine trip low auto stop oil pressure (Item 21), and
- 480V bus undervoltage (Items 28b and 35a) and alarm (Item 35c) relays.

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Since some of these channels also perform the additional function of providing control room indication and recording for post accident monitoring (PAM) functions, this application includes evaluations of the post accident monitoring function of the ESFAS circuits.

Other changes included in this application are: 1) the addition to Table 3.5-5 of limiting conditions for operation (LCO) requirements for the wide range containment pressure variable to ensure consistency with Regulatory Guide 1.97 commitments and the Indian Point 3 Emergency Operating Procedure (EOPs); 2) the addition of a quarterly functional test surveillance requirement to Item 4 of Table 4.1-1 for the low T_{avg} actuation circuits of the reactor coolant temperature channels; 3) the addition of a second line to item 14 of Table 4.1-1 to specify surveillance requirements for the wide range containment pressure instrumentation channels; and 4) the revision of item 20 to Table 4.1-1 to clarify that both the reactor trip and the engineered safety features actuation relay logic channels are functionally tested.

The proposed Technical Specification changes are contained in Attachment I to the Application for Amendment enclosed with this letter, and the associated Safety Evaluation is provided as Attachment II. The Authority's commitments associated with this application are presented in Attachment III.

A copy of this application and the associated attachments are being provided to the designated New York State official as required by 10 CFR 50.91.

If you have any questions, please contact Mr. P. Kokolakis.

Very truly yours,


W. A. Josiger
Acting Executive Vice President
Nuclear Generation

attachments: as stated
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