

A. Alan Blind
Vice President

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October 15, 1999

Re: Indian Point Station
Docket No. 50-247

Document Control Desk
US Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555

Dear Sir:

Enclosed is the Monthly Operating Report for Indian Point Unit No. 2 for
September 1999.

Very truly yours,

A. Alan Blind

Enclosure

cc: Mr. Hubert J. Miller
Regional Administrator - Region I
US Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector
US Nuclear Regulatory Commission
PO Box 38
Buchanan, NY 10511

Mr. Paul Eddy
State of New York Department of Public Service
3 Empire Plaza
Albany, NY 12223

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9910290095 990930
PDR ADOCK 05000247
R PDR

OPERATING DATA REPORT

DOCKET NO. 50-247

DATE October 6, 1999

COMPLETED BY F. Inzirillo

TELEPHONE (914)734-5179

OPERATING STATUS

Notes

1. Unit Name : Indian Point Unit #2
2. Reporting Period : September 1999
3. Licensed Thermal Power (Mwt) : 3071.4
4. Nameplate Rating (Gross Mwe) : 1008
5. Design Electrical Rating (Net Mwe) : 986
6. Maximum Dependable Capacity (Gross Mwe) : 965
7. Maximum Dependable Capacity (Net Mwe) : 931
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons :

9. Power Level To Which Restricted, If Any (Net Mwe) : _____
10. Reasons For Restrictions, If Any : _____

	This Month	Yr.-to-Date	Cumulative
11. Hours in Reporting Period	<u>720</u>	<u>6551</u>	<u>221352</u>
12. Number of Hours Reactor Was Critical	<u>0</u>	<u>5821.5</u>	<u>154941.12</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>4566.64</u>
14. Hours Generator On-Line	<u>0</u>	<u>5821.5</u>	<u>151266.78</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0</u>	<u>17632999</u>	<u>423050484</u>
17. Gross Electrical Energy Generated (MWH)	<u>0</u>	<u>5752511</u>	<u>131242217</u>
18. Net Electrical Energy Generated (MWH)	<u>-13700</u>	<u>5541110</u>	<u>125671300</u>
19. Unit Service Factor	<u>0</u>	<u>88.9</u>	<u>68.3</u>
20. Unit Availability Factor	<u>0</u>	<u>88.9</u>	<u>68.3</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0</u>	<u>90.2</u>	<u>64.0</u>
22. Unit Capacity Factor (Using DER Net)	<u>0</u>	<u>85.8</u>	<u>62.0</u>
23. Unit Forced Outage Rate	<u>100.0</u>	<u>11.1</u>	<u>11.3</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup : Unknown

	Forecast	Achieved
INITIAL CRITICALITY	<u>N/A</u>	<u>N/A</u>
INITIAL ELECTRICITY	<u>N/A</u>	<u>N/A</u>
COMMERCIAL OPERATION	<u>N/A</u>	<u>N/A</u>

AVERAGE DAILY POWER LEVEL

DOCKET NO. 50-247
 UNIT I.P. Unit #2
 DATE October 6, 1999
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 TELEPHONE (914)734-5179

MONTH September 1999

DAY	AVERAGE DAILY POWER LEVEL (Mwe-Net)	DAY	AVERAGE DAILY POWER LEVEL (Mwe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

INSTRUCTIONS

On this format, list the average daily unit power level in Mwe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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REPORT MONTH September 1999

No.	Date	Type(1)	Duration (Hours)	Reason(2)	Method of Shutting Down Reactor	Licensee Event Report #	System Code (4)	Component Code (5)	Cause & Corrective Action to Prevent Recurrence
1	990831	F	720	A	3	99-015-00	IA	(INSTRU) T	Reactor tripped due to Over Temp ΔT.

(1)
 F: Forced
 S: Scheduled

(2)
 Reason :
 A: Equipment Failure (Explain)
 B: Maintenance or Test.
 C: Refueling
 D: Regulatory Restriction
 E: Operator Training & License Examination
 F: Administrative
 G: Operational Error (Explain)
 H: Other (Explain)

(9/77)

(3)
 Method:
 1-Manual
 2- Manual Scram.
 3-Automatic Scram.
 4-Other Explain

(4)
 Exhibit G-Instructions for Preparation of
 Data Entry Sheets for Licensee Event
 Report (LER) File (NUREG-0161)

(5)
 Exhibit I - Same Source

SUMMARY OF OPERATING EXPERIENCE

September 1999

On August 31, 1999 the Instrument and Control (I&C) group was replacing a defective Pressurizer Low Pressure Trip bistable for protection channel 3. To support this effort the Over Temperature Delta Temperature (OTDT) channel trip for protection channel 3 was placed in the trip position. At approximately 1431 hours, a spurious electrical spike occurred on OTDT channel 4. This made up the "two out of four" reactor trip logic to cause an OTDT reactor trip. The August 31, 1999 reactor trip and subsequent plant response was reported with the submittal of Licensee Event Report LER 99-015-00. The unit remained shutdown throughout the month of September, for determination of the causes, electrical equipment assessment, repair of equipment and upgrade of procedures in response to the August 31, 1999 event.

MAJOR SAFETY RELATED MAINTENANCE

W.O. #	SYSTEM	COMPONENT	DATE COMPLETED	WORK PERFORMED
99-10837	IE	Channel 1 Ammonia Sensor	9/8/99	Replace sensor and perform channel calibration.
99-10716	ED	24 Battery	9/24/99	Performed slow charge of 24 battery, connected temporary charger and performed equalizer charge on cell #2
99-10706	EB	480 VAC Bus 6A	9/13/99	Performed high current testing on 23 EDG output breaker to troubleshoot Bus 6A condition..
99-10917	AB	Fire Protection piping	9/29/99	Ground out and repaired through wall weld defect on elbow near valve FP-45.
99-08867	EE	23 EDG	9/30/99	The over speed trip lever was resting on the trip pin. Adjusted the spring for the trip lever to position the pin as designed.