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Indian Point Station
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September 15, 1997

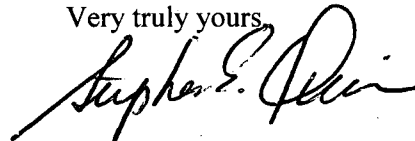
Re: Indian Point Station
Docket No. 50-247

Document Control Desk
US Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555

Dear Sir:

Enclosed is the Monthly Operating Report for Indian Point Unit No. 2 for August 1997.

Very truly yours,



Enclosure

cc: Mr. Hubert J. Miller
Regional Administrator - Region I
US Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector
US Nuclear Regulatory Commission
PO Box 38
Buchanan, NY 10511

Mr. Paul Eddy
State of New York Public Service Commission
Department of Public Service
3 Empire Plaza
Albany, NY 12223

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SUMMARY OF OPERATING EXPERIENCE

August 1997

Unit 2 began the month of August at 100% reactor power until August 6, 1997 at 2000 hrs. when a controlled shutdown was initiated pending confirmation that pressurizer safety valve settings were within allowable limits.

The unit was taken off the bus at 0200 hrs. on August 7, 1997 followed by a manual shutdown of the reactor at 0243 hrs. Upon re-adjustment of safety valve setpoints and re-installation the reactor was made critical on August 16, 1997 at 0250 hrs. followed by turbine synchronization on 8/17/97 at 0100 hrs. Power escalation reached 100% on August 18, 1997 at approximately 0130 hrs.

On August 21, 1997 at approximately 1530 hrs. power was reduced due to degraded feedwater isolation capability. At approximately 1800 hrs. with reactor power at <50% adjustments were made to #22 Main Boiler Feed pump discharge valve. Power escalation commenced, reaching 100% on August 22, 1997 at 1020 hrs.

On August 23, 1997 a reactor trip occurred at 1625 hrs. due to failure of an RCP test relay. Upon replacement of appropriate and similar relays on all reactor coolant pumps the reactor was made critical on August 25, 1997 at 0636 hrs. followed by turbine synchronization at 1318 hrs.

Reactor power was increased to 100% by 1300 hrs. on August 26, 1997 and remained steady through month's end.

OPERATING DATA REPORT

DOCKET NO. 50-247

DATE Sept. 10, 1997

COMPLETED BY F. Inzirillo

TELEPHONE (914)734-5179

OPERATING STATUS

Notes

1. Unit Name : Indian Point Unit #2
 2. Reporting Period : August 1997
 3. Licensed Thermal Power (Mwt) : 3071.4
 4. Nameplate Rating (Gross Mwe) : 1008
 5. Design Electrical Rating (Net Mwe) : 986
 6. Maximum Dependable Capacity (Gross Mwe) : 965
 7. Maximum Dependable Capacity (Net Mwe) : 931

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons :

9. Power Level To Which Restricted, If Any (Net Mwe) : _____
 10. Reasons For Restrictions, If Any : _____

	This Month	Yr.-to-Date	Cumulative
11. Hours in Reporting Period	<u>744</u>	<u>5831</u>	<u>203112</u>
12. Number of Hours Reactor Was Critical	<u>489.70</u>	<u>2757.98</u>	<u>145303.50</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>131.87</u>	<u>4566.64</u>
14. Hours Generator On-Line	<u>460.12</u>	<u>2582.73</u>	<u>141689.63</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>1330973</u>	<u>7290791</u>	<u>394269078</u>
17. Gross Electrical Energy Generated (MWH)	<u>415336</u>	<u>2319409</u>	<u>121858167</u>
18. Net Electrical Energy Generated (MWH)	<u>393377</u>	<u>2188947</u>	<u>116682721</u>
19. Unit Service Factor	<u>61.8</u>	<u>44.3</u>	<u>69.8</u>
20. Unit Availability Factor	<u>61.8</u>	<u>44.3</u>	<u>69.8</u>
21. Unit Capacity Factor (Using MDC Net)	<u>56.8</u>	<u>40.0</u>	<u>65.1</u>
22. Unit Capacity Factor (Using DER Net)	<u>53.6</u>	<u>38.1</u>	<u>63.2</u>
23. Unit Forced Outage Rate	<u>38.2</u>	<u>37.2</u>	<u>6.9</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup : _____

26. Units In Test Status (Prior to Commercial Operation) :

	Forecast	Achieved
INITIAL CRITICALITY	<u>N/A</u>	<u>N/A</u>
INITIAL ELECTRICITY	<u>N/A</u>	<u>N/A</u>
COMMERCIAL OPERATION	<u>N/A</u>	<u>N/A</u>

AVERAGE DAILY POWER LEVEL

DOCKET NO. 50-247

UNIT I.P. Unit #2

DATE 09/10/97

COMPLETED BY F. Inzirillo

TELEPHONE (914)734-5179

MONTH August 1997

DAY AVERAGE DAILY POWER LEVEL
(Mwe-Net)

1	<u>932</u>
2	<u>935</u>
3	<u>936</u>
4	<u>936</u>
5	<u>937</u>
6	<u>865</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY AVERAGE DAILY POWER LEVEL
(Mwe-Net)

17	<u>437</u>
18	<u>915</u>
19	<u>928</u>
20	<u>930</u>
21	<u>787</u>
22	<u>835</u>
23	<u>625</u>
24	<u>0</u>
25	<u>98</u>
26	<u>825</u>
27	<u>932</u>
28	<u>934</u>
29	<u>936</u>
30	<u>937</u>
31	<u>937</u>

INSTRUCTIONS

On this format, list the average daily unit power level in Mwe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

DOCKET NO. 50-247
 UNIT I.P. Unit #2
 DATE 09/10/97
 COMPLETED BY F. Inzirillo
 TELEPHONE (914)734-5179

REPORT MONTH AUGUST 1997

No.	Date	Type(1)	Duration (Hours)	Reason(2)	Method of Shutting Down Reactor	Licensee Event Report #	System Code (4)	Component Code (5)	Cause & Corrective Action to Prevent Recurrence
N/A	97 08 06	F	0	H	4		CC	(ValveX) G	Unit being removed from service due to Safety Relief Valve setpoints not conforming to Technical Specifications.
5	97 08 07	F	239.00	H	1		CC	(ValveX) G	Unit removed from service to re-calibrate setpoints on Pressurizer Safety Relief Valves. Reactor was manually shutdown.
6	97 08 23	F	44.88	A	3		IA	(RelayX) X	Unit trip occurred due to a failure of a Test Relay from 23 Reactor Coolant Pump

(1)
 F: Forced
 S: Scheduled

(2)
 Reason :
 A: Equipment Failure (Explain)
 B: Maintenance or Test
 C: Refueling
 D: Regulatory Restriction
 E: Operator Training & License Examination
 F: Administrative
 G: Operational Error (Explain)
 H: Other (Explain)

(3)
 Method:
 1-Manual
 2- Manual Scram.
 3-Automatic Scram.
 4-Other Explain

(4)
 Exhibit G-Instructions for Preparation of
 Data Entry Sheets for Licensee Event
 Report (LER) File (NUREG-0161)

(5)
 Exhibit I - Same Source

(9/77)

MAJOR SAFETY RELATED MAINTENANCE

W.O. #	SYSTEM	COMPONENT	DATE COMPLETED	WORK PERFORMED
93728	RCS	PCV-464	8/12/97	Removed valves, sent to Wyle Labs and replaced.
93482	RCS	PCV-466	8/12/97	Removed valves, sent to Wyle Labs and reinstalled.
93729	RCS	PCV-468	8/12/97	Removed valves, sent to Wyle Labs and reinstalled.
94029	RPS	RPSA	8/24/97	Replaced Relays
94030	RPS	RPSA	8/24/97	Replaced Relays
94031	RPS	RPSA	8/24/97	Replaced Relays
94032	RPS	RPSA	8/24/97	Replaced Relays
94033	RPS	RPSA	8/24/97	Replaced Relays