James S. Baumark
Vice President
Nuclear Engineering

Consolidated Edison Company of New York, Inc. Indian Point 2 Station Broadway & Bleakley Avenue Buchanan, New York 10511

Internet: baumstarkj@coned.com Telephone: (914) 734-5354 Cellular: (914) 391-9005 Pager: (917) 457-9698 Fax: (914) 734-5718 June 24, 1999

Re:

Indian Point Unit No. 2 Docket No. 50-247

LER 97-17-01

Document Control Desk US Nuclear Regulatory Commission Mail Station PI-137 Washington, DC 20555

The attached Licensee Event Report 97-17-01 is hereby submitted in accordance with the requirements of 10 CFR 50.73.

Very truly yours,

Attachment

C: Mr. Hubert J. Miller
Regional Administrator - Region I
US Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. Jefferey Harold, Project Manager Project Directorate I-1 Division of Reactor Projects I/II US Nuclear Regulatory Commission Mail Stop 14B-2 Washington, DC 20555

Senior Resident Inspector US Nuclear Regulatory Commission PO Box 38 Buchanan, NY 10511

9907080117 990624 PDR ADOCK 05000247 PDR

NRC FO (6-1998)	PRM 36	LICE (S	ee reve	rse	U.S. NUCL /ENT REF for required acters for each	PORT	- (L	.ER)	COMM	MISSION	ECEDRESC	stimate ollection censing urden e tegulate teduction Vashing urrently	ed burden n request: ! g process estimate to bry Commis on Projec gton, DC valid OME	and fed back to	comply with essons learn industry. For gement Bray , DC 20555 Office of to commation commeted to the commeted	this mand ned are incomment con nnch (T-6 F -0001, and Manageme ollection do y not condu	33), U.S. Nuclear to the Paperwork ent and Budget, des not display a lict or sponsor, and	
FACILITY	NAME	(1)								· · · · · · · · · · · · · · · · · · ·	D	DOCKET NUMBER (2)				PAGE (3)		
Indian	Point	No. 2							٠	;		. 0!	5000-2	47		1	OF 4	
TITLE (4)										•								
Reviev	v of r	nonthly i	radiatio	n n	nonitor perf	orma	nce	test da	ata.							•		
EVE	NT DA	TE (5)		LER NUMBER (6)					PORT D	PATE (7	' }				CILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR		SEQUENTIAL Number	REVISI NUMB		MONTH	DAY	YE	AR	FA	CILITY NAM	ЛЕ .		0500 0500		
7	21	1997	199	7	17	1		6	24	19	99	FACILITY NAME			DOCKET NUMBER 05000			
OPERA	TING	N				SUBM	Ш				REC			OF 10 CFR 5:	(Check or			
MODE (9)		l 'N	20.2201(b)			20.2203(a)(2)(v)				X 50.73(a)(2)(i)				50.73(a)(2)(viii)				
POWER LEVEL (10)		90	\vdash	20.2203(a)(1) 20.2203(a)(2)(i) 20.2203(a)(2)(ii)			20.2203(a)(3)(ii) 20.2203(a)(3)(iii) 20.2203(a)(4)				+	50.73(a)(2)(ii)			50.73(a)(2)(x) 73.71			
		<u> </u>	ⅎ⊢								50.73(a)(2)(iii) 50.73(a)(2)(iv)				OTHER			
			20.2203(a)(2)(iii) 20.2203(a)(2)(iii)				50.36(c)(1)				50.73(a)(2)(v)			Specify in Abstract below				
			∥	20.2203(a)(2)(iv)			50.36(c)(2)			•	50.73(a)(2)(vii)				or in NRC Form 366A			
			<u> </u>			LIC	CENS	SEE CON	TACT	OR TH	IS L	ER (1:	2)					
NAME														UMBER (Include Ar	ea Code)			
John Beck, Senior Licensing Engineer									(914) 734-5692									
			CO	MPL	ETE ONE LINE	FOR I	ĘAC	н сомро	DNENT	FAILUF	RE D	ESCR	IBED IN	THIS REPORT	(13)			
CAUSE		SYSTEM	COMPONENT MANUFACTURER			REPORTABLE CAUSE		JSE		SYSTEM	COMPONENT	MANUFA	CTURER	REPORTABLE TO EPIX				
													: •					
															• •			
		<u> </u>	SUPI	LEME	NTAL REPORT E	XPECTE	0 (14)						EA	PECTED	MONTH	DAY	YEAR	
YE (If		omplete EX	(PECTE	SU	BMISSION DA	TE).			X	0			SUB	MISSION ATE (15)		,		

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On July 21, 1997, with the unit at approximately 90 percent power, the results of a radiation monitoring system performance test were reviewed and determined to be unsatisfactory. The failure of performance test PT-M86 is attributed to a calibration constant being set incorrectly. The calibration constant is used to calculate the gaseous radioactivity in the in-service Large Gas Decay Tank that is then compared against the alarm set point. Evaluation of the results indicated that the calculated activity value of Large Gas Decay Tank gaseous monitor (R-50) activity was nonconservatively low. Once noticed, the calibration constant was restored to the proper value and, subsequently, the radiation monitor was declared operable.

NRC FORM 366A

(6-1998)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET (2)		LER NUMBER (6)	PAGE (3)
	05000 047	YEAR	SEQUENTIAL NUMBER	REVISION Number	2 05 4
Indian Point No. 2	05000-247	1997	017	01	2 OF 4

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

PLANT AND SYSTEM IDENTIFICATION:

Westinghouse 4-Loop Pressurized Water Reactor

IDENTIFICATION OF OCCURRENCE:

Review of monthly radiation monitor performance test data.

EVENT DATE:

July 21, 1997

REPORT DUE DATE:

August 20, 1997

REFERENCES:

Condition Identification and Tracking System (CITRS) No. 97-E02792 and 97-E02794

PAST SIMILAR OCCURRENCE:

None

DESCRIPTION OF OCCURRENCE:

On July 21, 1997, at approximately 15:36 hours, the results of the R-50 radiation monitor performance test were reviewed and found to be unacceptable. A work order was initiated to implement corrective maintenance on R-50 to repair the cause of the low reading. The calibration constant was restored to the required value. At 17:25 hours, on July 21, 1997, R-50 was declared operable.

During the time that the calibration constant was set nonconservatively, the Large Gas Decay Tank (LGDT) radiation monitor, R-50, was inoperable.

NRC FORM 366A

(6-1998)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET (2)		LER NUMBER (6)	PAGE (3)
	05000 047	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	0 05 4
Indian Point No. 2	05000-247	1997	017	01	3 OF 4

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

ANALYSIS OF OCCURRENCE:

During the performance of monthly test PT-M86, for radiation monitor R-50, Large Gas Decay Tank (LGDT) activity, it was discovered that the conversion constant was nonconservatively set at 9.0E-15 micro Ci/cc/Amp. The correct value for this application is 7.22E+8 micro Ci/cc/Amp. This specific conversion constant is used to calculate the radioactive activity for the in service Large Gas Decay Tank. During the time that the conversion constant was set incorrectly at 9.0 E-15 micro Ci/cc/Amp, the R-50 radiation monitor was inoperable. The Technical Specification (TS) requirement for minimum number of Waste Hold Up System noble gas activity monitor operable channels was not met, nor was the compensatory action of performing daily determinations of receiving gas decay tank activity. The TS and administrative limits for LGDT activity are 29761 and 6000 Curies respectively. The R-50 set point of 600 Curies provides an alarm function only. Automatic control functions are not provided. Review of periodic analysis of the manually sampled LGDT indicates there were no significant amounts (greater than one Curie) of LGDT activity during the period that the R-50 radiation monitor calibration constant was incorrectly set.

These events are reportable under 10 CFR 50.73(a)(2)(i)(b). The basis for reportability is that the Technical Specification 3.9.B.2.a requirement for radioactive gaseous effluent monitoring instrumentation was not met. Further, the compensatory action to sample tank activity was not performed on a daily basis.

CAUSE OF OCCURRENCE:

Our identification of the discrepancy and subsequent review of digital radiation monitoring system one hour interval data printouts from July 9, 1997 indicate that the conversion constant was changed sometime between 10:00 and 11:00 hours of that day. This review of printouts revealed a dramatic reduction in hourly average activity from 1.15E-2 to 4.54E-25 Ci. During the time in question, the radiation monitor system engineer and a nuclear plant operator (NPO) were troubleshooting unrelated operational difficulties with R-50. These troubleshooting activities focused on "high pressure" alarms. During their troubleshooting activities, R-50 was placed into local control to verify programmed set points.

The cause of this event is attributed to personnel error. This is based on failure to realize the calibration constant was manipulated. The time delay for radiation monitor display update was not sufficiently understood by involved parties

NRC	FORM	366A
(6-1998	3)	

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET (2)		LER NUMBER (6) .	PAGE (3)
Ladica Deiga No. O	05000 047	YEAR	SEQUENTIAL NUMBER	REVISION Number	4 05 4
Indian Point No. 2	05000-247	1997	017	01	4 OF 4

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

CORRECTIVE ACTIONS:

The calibration constant was restored to the value specified in Test and Performance Procedure PT-M86 for the radiation monitoring system.

Management's expectations were reinforced with the Nuclear Plant Operators (NPO's) regarding the use of procedures when operating equipment. The operation of radiation monitors was discussed, together with the importance of error codes and their functions.

Procedure PT-M86 has been revised to require recording the "as-found" LCU display reading prior to changing any key switch positions. This will aid in troubleshooting of radiation monitors and in any follow-up investigation.

Procedure PT-D5 has been revised to reference an operator aid containing the lower boundary values to identify a potentially abnormal reading (the upper boundary value is the alarm set point).