A. Alan Blind
Vice President

Consolidated Edison Company of New York, Inc.
Indian Point Station
Broadway & Bleakley Avenue
Buchanan, NY 10511
Telephone (914) 734-5340
Fax: (914) 734-5718
blinda@coned.com

October 23, 1998

Re: Indian Point Unit No. 2
Docket No. 50-247
LER 98-016-00

Document Control Desk
US Nuclear Regulatory Commission
Mail Station PI-137
Washington, DC 20555

The attached Licensee Event Report 98-016-00 is hereby submitted in accordance with the requirements of 10 CFR 50.73.

Very truly yours,

A. Alan Blind

Attachment

cc: Mr. Hubert J. Miller
Regional Administrator - Region I
US Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. Jefferey Harold, Project Manager
Project Directorate I-1
Division of Reactor Projects I/II
US Nuclear Regulatory Commission
Mail Stop 14B-2
Washington, DC 20555

Senior Resident Inspector
US Nuclear Regulatory Commission
PO Box 38
Buchanan, NY 10511

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NRC FORM 366 (6-1998) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104 EXPIRES 06/30/2001

Estimated burden per response to comply with this mandatory information collection request: 50 hrs. Reported lessons learned are incorporated into the licensing process and fed back to industry. Forward comments regarding burden estimate to the Records Management Branch (T-6 F33), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to the Paperwork Reduction Project (3150-0104), Office of Management and Budget, Washington, DC 20503. If an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Indian Point No. 2	05000-247	1 OF 3

TITLE (4)

Inadequate Compliance with Technical Specification Required Action for Inoperable Rod Position Deviation Alarm

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
09	21	1998	1998	-- 016 --	00	10	21	1998		05000
									FACILITY NAME	DOCKET NUMBER
										05000

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)			
POWER LEVEL (10)		20.2201(b)	20.2203(a)(2)(v)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)
		20.2203(a)(1)	20.2203(a)(3)(i)	50.73(a)(2)(ii)	50.73(a)(2)(x)
		20.2203(a)(2)(i)	20.2203(a)(3)(ii)	50.73(a)(2)(iii)	73.71
		20.2203(a)(2)(ii)	20.2203(a)(4)	50.73(a)(2)(iv)	OTHER
		20.2203(a)(2)(iii)	50.36(c)(1)	50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A
		20.2203(a)(2)(iv)	50.36(c)(2)	50.73(a)(2)(vii)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
James J. Maylath, Senior Engineer	(914) 734-5356

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX
A	ID	CPU	W120	N					

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE).	X	NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On October 3, 1998, the input to the rod position deviation alarm was found in the 'disable' position which rendered the alarm inoperable. This alarm is used for monitoring rod position deviation, and its input had been in the 'disable' position since September 20, 1998. Technical Specification required actions for an inoperable rod position deviation alarm include logging individual rod positions once per shift and after a load change greater than 10%. Individual rod positions are normally printed out and logged twice per shift, and this action was done during the period the rod position deviation alarm was in the 'disable' position. However, on September 21, 1998, during startup of the unit, additional logging of individual rod positions was not performed following the load change greater than 10%, contrary to Technical Specification Section 3.10.9. The rod position deviation alarm was restored after it was found in the 'disable' position.

FACILITY NAME (1)	DOCKET (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Indian Point No. 2	05000-247	1998	-- 016 --	00	2 OF 3

TEXT *(If more space is required, use additional copies of NRC Form 366A)* (17)

PLANT AND SYSTEM IDENTIFICATION:

Westinghouse 4-Loop Pressurized Water Reactor

IDENTIFICATION OF OCCURRENCE:

Inadequate Compliance with Technical Specification Required Action for Inoperable Rod Position Deviation Alarm

EVENT DATE:

September 21, 1998

REPORT DUE DATE:

October 21, 1998

REFERENCES:

Condition Reporting System (CRS) No. 199808768

PAST SIMILAR OCCURRENCE:

LER 1986-039

DESCRIPTION OF OCCURRENCE:

On October 3, 1998, the computer input for the rod position deviation alarm was found in the 'disable' position. The plant computer monitors rod position deviation and generates this alarm for deviations greater than an established value. With the computer input in the 'disable' position, the rod position deviation alarm was inoperable. This computer input had been in the 'disable' position since September 20, 1998. Section 3.10.9 of the Technical Specification requires logging individual rod positions once per shift and after a load change greater than 10% if the rod position deviation alarm is inoperable. Individual rod positions are normally printed out by the plant computer and logged twice per shift. Rod positions were printed out and logged twice per shift during the period the rod position deviation alarm was in the 'disable' position. However, on September 21, 1998, during startup of the unit, the additional logging of individual rod positions was not performed following the load change greater than 10%, contrary to Technical Specification Section 3.10.9.

FACILITY NAME (1)	DOCKET (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Indian Point No. 2	05000-247	1998	-- 016 --	00	3 OF 3

TEXT *(If more space is required, use additional copies of NRC Form 366A)* **(17)**

ANALYSIS OF OCCURRENCE:

This report is being submitted in accordance with 10CFR50.73(a)(2)(i) for plant operation in a condition that is prohibited by Technical Specifications. Section 3.10.9 of the Technical Specification requires logging individual rod positions once per shift and after a load change greater than 10% if the rod position deviation alarm is inoperable. Although individual rod positions were printed out and logged twice per shift, the additional logging of individual rod positions was not performed following the load change greater than 10% on September 21, 1998.

The intent of the additional logging, if the rod position deviation alarm is inoperable, is to provide for additional operator awareness of rod position and allow for timely operator response to a rod deviation. The operators did not have this additional awareness during the plant startup on September 21, 1998. A review of the plant computer data did not indicate any significant rod deviation. This event did not have any impact on plant safety. There were no personnel injuries or damage to equipment as a result of this event.

CAUSE OF OCCURRENCE:

The cause of the failure of the operators to perform additional logging of individual rod positions following the load change greater than 10% on September 21, 1998, was that the rod position deviation alarm computer input was inadvertently disabled on September 20, 1998, when operators took action they believed would reset the computer point, which was in the 'alarm' state. The deviation alarm had not cleared after the input for failed individual rod position indication, IRPI C07, was disabled. This led operators to take action to reset the alarm by means of the plant computer. The operators had inadequate procedural guidance on the details of using the plant computer, and Computer Application Section personnel should have been contacted prior to taking action on the computer point. The cause of the disabling of the rod position deviation alarm computer input is therefore attributed to human error because the operators took action without questioning the appropriate personnel despite the lack of procedural guidance.

CORRECTIVE ACTION:

The rod position deviation alarm computer input was restored (returned to the plant computer scan) following the discovery of it being disabled on October 3, 1998. Plant computer data was reviewed to determine rod positions and reactor power changes during the period when the additional logging was required and when the rod position deviation alarm computer input was disabled (removed from the plant computer scan).

The operators will receive training on this event and on proper use of the plant computer for applicable alarms. This training will be completed by December 15, 1998. Appropriate procedural guidance will be developed on the manipulation of data and alarm inputs on the plant computer. This procedural guidance will be developed by December 15, 1998.