Paul H. Kinkel
Vice President

Consolidated Edison Company of New York, Inc.
Indian Point Station
Broadway & Bleakley Avenue
Buchanan, NY 10511
Telephone (914) 734-5340
Fax: (914) 734-5923

April 20, 1997

Re: Indian Point Unit No. 2
Docket No. 50-247
LER 98-005-00

Document Control Desk
US Nuclear Regulatory Commission
Mail Station PI-137
Washington, DC 20555

The attached Licensee Event Report 98-005-00 is hereby submitted in accordance with the requirements of 10 CFR 50.73.

Very truly yours,

Paul H Kinkel

Attachment

cc: Mr. Hubert J. Miller
Regional Administrator - Region I
US Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. Jefferey Harold, Project Manager
Project Directorate I-1
Division of Reactor Projects I/II
US Nuclear Regulatory Commission
Mail Stop 14B-2
Washington, DC 20555

Senior Resident Inspector
US Nuclear Regulatory Commission
PO Box 38
Buchanan, NY 10511

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NRC FORM 366 (6-89)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3150-104
EXPIRES: 4/30/92

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Indian Point No. 2	0 5 0 0 0 2 4 7	1 OF 0 3

TITLE (4)

Common Power Supply Found on Pressurizer Power-Operated Relief Valves

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 3	2 0	9 8	9 8	- 0 0 5 -	0 0	0 4	2 0	9 8		0 5 0 0 0
										0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR§: *(Check one or more of the following)* (11)

OPERATING MODE (9)	N								
POWER LEVEL (10)	0 0 0	20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)
		20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)
		20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER *(Specify in Abstract below and in Text, NRC Form 336A)*
		20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		
		20.405(a)(1)(iv)	X	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
		20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (AREA CODE)
James J. Maylath, Senior Engineer	9 1 4 7 3 4 - 5 3 5 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X YES *(If yes, complete EXPECTED SUBMISSION DATE)*	NO		0 9	3 0	9 8

ABSTRACT *(Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines)* (16)

On March 20, 1998, with the unit at cold shutdown, the Power-Operated Relief Valves (PORVs), PCV-455C and PCV-456, were found to have a common 125 VDC power source for both solenoids and control circuits. This condition could have rendered the Overpressure Protection System (OPS) inoperable if a failure of the common 125 VDC power supply were to occur. The discovery was made of a jumper in the Central Control Room Flight Panel FBF. Subsequently, it was verified that this jumper was not in the plant drawings and the power supply, which should have been connected to PCV-455C, as shown in the plant drawings, was disconnected upstream of the supply fuses. At this time, a review of the Jumper Log is being conducted, and all physical evidence (e.g. - photographs, jumper wires, documentation of prior inspections, etc.) are being preserved to support a full investigation of the source and timing of this "jumper" assuming documentation of this event is not found in the official Jumper Log.

PLANT AND SYSTEM IDENTIFICATION:

Westinghouse 4-Loop Pressurized Water Reactor

IDENTIFICATION OF OCCURRENCE:

Common Power Supply Found on Pressurizer Power-Operated Relief Valves

EVENT DATE:

March 20, 1998

REPORT DUE DATE:

April 20, 1998

REFERENCES:

Condition Identification and Tracking System (CITRS) No. 98-E02240

PAST SIMILAR OCCURRENCE:

None.

DESCRIPTION OF OCCURRENCE:

On March 20, 1998, a voluntary power supply verification was initiated to demonstrate that the 125 VDC and Instrument bus circuitry and load were consistent with the expected plant configuration. During the performance of Test Procedure IC-SL-61 for 125 VDC Distribution Panel 22, circuit 15, the green position indicating light for valve PCV-455C did not extinguish when the valve switch was placed in the closed position. According to plant drawings PCV-455C is fed off circuit 15 of Distribution Panel 22. When the control switch for PCV-455C was placed in the open position with circuit 15 open, the green light remained lit.

A field check of the wiring for Distribution Panel 22, circuit 15, and Distribution Panel 21, circuit 5 (which feeds PCV-456), established that the two circuits were cross connected at the fuse clips for PCV-456 and PCV-455C behind the central control room FBF panel. The power supply for PCV-455C (as per plant drawings)

was found disconnected from the upstream side of Fuses FU22P & N with the lugged ends taped and hanging in the vicinity of the fuses; the other end of this required supply line was still connected to the terminal block in panel FBF. Closer inspection of the fuse block uncovered a tag tied to both jumper wires which read:

"REMOVE JUMPER AND RECONNECT TO P1-N1 STRIP L2"

Initially, the tag had not been found because it was hanging behind the fuses, which are not flush mounted. The tag is of white manila with a brown tie ring, similar to other wire tags in the vicinity, but does not have any date or indication of responsible individual. The tag is not of a type currently used by Operations in conjunction with control and administration of the Jumper Log.

ANALYSIS OF OCCURRENCE:

This report is being submitted in accordance with 10CFR50.73(a)(2)(ii) for a condition that is outside the plant design basis. This plant condition, as discovered, could have resulted in a single failure to the common 125 VDC supply rendering the Overpressure Protection System inoperable. At the time of discovery the unit was in cold shutdown with the RCS temperature <305°F, and the OPS was not required to be operable. There was no injury to personnel, nor was there any damage to equipment.

CAUSE OF OCCURRENCE:

The cause of this event leading to a plant condition being outside designed basis is still under investigation. Final determination of the cause of this event is pending completion of various document reviews and will be provided in a supplement to this report, tentatively scheduled for submittal on or before September 30, 1998.

CORRECTIVE ACTION:

Within an hour of the discovery of both PORVs being powered from a common bus, notification was made in accordance with 10CFR50.72(b)(1)(ii)(B). Since the plant was in cold shutdown the OPS was not required to be operable and no immediate action was initiated with regard to re-establishing the design basis wiring configuration. The design basis wiring configuration has now been restored. With the initiation of a full investigation, final corrective actions will be determined based upon investigation results and analysis.