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April 9, 1998

Re: Indian Point Unit No. 2
Docket No. 50-247

Document Control Desk
US Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555-0001

Subject: Response to Concerns Regarding the Prairie Island Unit No. 2 Part
Length Control Rod Mechanism Leakage Issue

In a letter to Mr. Frank J. Miraglia, Deputy Director, Office of Nuclear Reactor Regulation, dated March 6, 1998, (OG-98-037) the Westinghouse Owners Group (WOG) responded to the NRC's formal request to activate the Regulatory Response Group (RRG) and address concerns regarding the generic implications of the part length control rod drive mechanism (CRDM) housing issue identified at the Prairie Island Unit 2. That letter documented the information provided by the WOG during a meeting with the NRC on February 27, 1998. At that meeting the WOG presented information regarding the current assessment of the CRDM housing issue and the activities planned to continue to resolve the potential generic concerns. The condition assessment report provided to NRC emphasized that the incident is an isolated one, and that it is unlikely to occur in other housings. Furthermore, even if it were to occur, and it led to severance of a part length housing, the event has already been considered as part of each plant's design basis.

The affected WOG member utilities met on March 2, 1998 to review and discuss the same information presented to the NRC. The outcome of this meeting was the formation of a subgroup that will continue to support Westinghouse member utilities in responding to this issue. From the discussions at that meeting, the WOG RRG recommended that the affected utilities voluntarily consider notifying the NRC of their plans to address this issue. For plants currently in refueling or cold shutdown, it was recommended that the utilities docket their plans to address this issue within 30 days, or prior to startup. For those units continuing to operate with part length CRDM housings installed, it was further recommended that the power plant operators increase reactor coolant system leakage monitoring awareness and identify how this is being accomplished in their individual response letters.

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Consolidated Edison Company of New York, Inc.'s (Con Edison) recently completed actions that addressed the CRDM housing issue at Indian Point Unit No. 2. Con Edison has closely followed the Prairie Island part length control rod mechanism leakage issue through it's participation with the WOG. To preclude the possibility of a similar failure of a part length control rod mechanism at Indian Point Unit 2, Con Edison removed the mechanisms and replaced them with caps. This modification was performed on all eight (8) part length control rod mechanisms during the current maintenance outage. All removed mechanisms will be retained by Con Edison for future investigations.

Should you or your staff have any concerns regarding this matter, please contact Mr. Charles W. Jackson, Manager, Nuclear Safety & Licensing.

Very truly yours,

Paul Michel

C: Mr. Hubert J. Miller
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