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November 30, 1994

Re: Indian Point Unit No. 2
Docket No. 50-247

Document Control Desk
US Nuclear Regulatory Commission
Mail Station P1-137
Washington, DC 20555

SUBJECT: 30 Day Report Pursuant to 10 CFR 50.46 (3)(ii)

This is to inform you that our vendor, Westinghouse, has notified us via Nuclear Safety Advisory Letter NSAL-94-022M that they have completed an evaluation of a potential issue with respect to the axial nodalization used in the SBLOCTA code, which is a part of the Small Break Emergency Core Cooling System evaluation model for Indian Point 2. In addition, they have completed their revised model for calculating transient fuel rod internal pressure in the SBLOCTA code. Several recent code revisions and error corrections of lesser magnitude have also been incorporated in the SBLOCTA code. The result of the above evaluation was a 303 degree F decrease in the peak clad temperature (PCT) for IP2. Previously, Westinghouse had completed an evaluation of the effect of the containment spray during a small break LOCA and they had corrected a boiling heat transfer correlation error and a steam line isolation logic error. The net effect of these was a 32 degree F increase in PCT. Although the effect of all of the above is a net decrease in the calculated small break PCT, we are reporting this to you since the sum of the absolute magnitudes of the respective PCT changes is greater than 50 degrees F.

Westinghouse has informed us that they have notified you of the resolution of all the above issues.

Should you have any questions regarding this matter, please contact Mr. Charles W. Jackson, Manager, Nuclear Safety and Licensing.

Very truly yours,



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cc:

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