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James Knubel
Senior Vice President and
Chief Nuclear Officer

April 5, 1999
IPN-99-034

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Subject: **Indian Point 3 Nuclear Power Plant**
Docket No. 50-286
1998 Emergency Core Cooling System Evaluation Changes

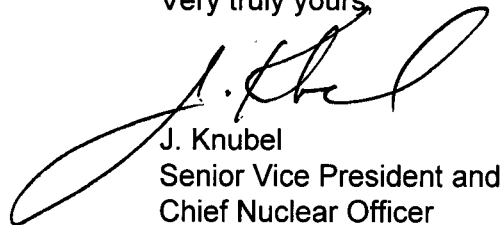
- References:
1. Westinghouse letter (INT-99-211), S. Ira to R. Barrett, "10 CFR 50.46 Annual Notification and Reporting for 1998," dated March 5, 1999.
 2. NYPA letter to NRC (IPN-97-106), "Emergency Core Cooling System Evaluation Changes," dated August 7, 1997.

Dear Sir:

The Authority has reviewed the 1998 small and large break loss of coolant accident (LOCA) peak cladding temperature (PCT) changes described in Reference 1. There are no additional changes to the small or large break PCT since the submittal of Reference 2. Therefore, the Indian Point 3 PCT remains below the maximum fuel cladding temperature of 2200°F, and Indian Point 3 continues to comply with 10 CFR 50.46. This letter satisfies the annual reporting requirements of 10 CFR 50.46(a)(3)(ii).

No commitments are being made by the Authority in this submittal. If you have any questions, please contact Ms. C. D. Faison.

Very truly yours,


J. Knubel
Senior Vice President and
Chief Nuclear Officer

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